

JENDL-4.0: A New Library for Nuclear Science and Engineering

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Citation Report

#	ARTICLE	IF	CITATIONS
1	Development of two-dimensional differential calibration method for a neutron dosimeter using a thermal neutron beam. , 2010, , .		0
2	Analysis of Core Physics Experiments on Fresh and Irradiated PWR UO ₂ Fuels in the REBUS Program. Journal of Nuclear Science and Technology, 2011, 48, 1025-1045.	0.7	5
3	Burnup Characteristics of a Peu À Peu Fuel Loading Scheme in a 110MWt Simplified Pebble Bed Reactor. Journal of Nuclear Science and Technology, 2011, 48, 1385-1395.	0.7	14
4	<sup>4</sup>He detectors for Mixed Oxide (MOX) fuel measurements. , 2011, , .		3
5	Neutron Nuclear Data Evaluation of Cesium Isotopes for JENDL-4.0. Journal of Nuclear Science and Technology, 2011, 48, 1238-1249.	0.7	7
6	Single Event Effects in Power MOSFETs and SRAMs Due to 3 MeV, 14 MeV and Fission Neutrons. IEEE Transactions on Nuclear Science, 2011, 58, 952-959.	1.2	41
7	Detailed benchmark test of JENDL-4.0 iron data for fusion applications. Fusion Engineering and Design, 2011, 86, 2682-2685.	1.0	5
8	Simulation of displacement damage for silicon avalanche photo-diodes. Nuclear Instruments and Methods in Physics Research, Section A: Accelerators, Spectrometers, Detectors and Associated Equipment, 2011, 658, 70-72.	0.7	0
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10	TRU burning by dual tier system of LWR-SFR. Progress in Nuclear Energy, 2011, 53, 902-908.	1.3	5
11	JENDL-4.0 Benchmarking for Effective Delayed Neutron Fraction of Fast Neutron Systems. Journal of Nuclear Science and Technology, 2011, 48, 1471-1477.	0.7	9
12	Measurement of the neutron capture cross-section of ²³² Th using the neutron activation technique. European Physical Journal A, 2011, 47, 1.	1.0	26
13	Measurement of the neutron-induced fission cross-section of ²⁴³ Am relative to ²³⁵ U from 0.5 to 20 MeV. European Physical Journal A, 2011, 47, 1.	1.0	11
14	Transmutation by adiabatic resonance crossing experiment (TARC) benchmarking. Annals of Nuclear Energy, 2011, 38, 2180-2186.	0.9	4
15	Measurement of Charged-Particle Emission Double-Differential Cross Section of Fluorine for 14.2MeV Neutrons. Journal of Nuclear Science and Technology, 2011, 48, 1146-1157.	0.7	3
16	JENDL-4.0 Benchmarking for Fission Reactor Applications. Journal of Nuclear Science and Technology, 2011, 48, 172-187.	0.7	58
17	Radiogenic Lead with Dominant Content of ²⁰⁸ Pb: New Coolant and Neutron Moderator for Innovative Nuclear Facilities. Science and Technology of Nuclear Installations, 2011, 2011, 1-12.	0.3	10
18	Advanced Method for Calculations of Core Burn-Up, Activation of Structural Materials, and Spallation Products Accumulation in Accelerator-Driven Systems. Science and Technology of Nuclear Installations, 2012, 2012, 1-12.	0.3	26

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20	Evaluation of neutron nuclear data for erbium. Journal of Nuclear Science and Technology, 2012, 49, 824-835.	0.7	6
21	Neutrino induced reactions for ν -process nucleosynthesis of ^{92}Nb and ^{98}Tc . Physical Review C, 2012, 85, .	1.1	42
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130	$\frac{1}{\sigma_{\text{Zr}}} = \frac{1}{\sigma_{\text{Tj}}} + \frac{1}{\sigma_{\text{ETQq1}}} + \frac{1}{\sigma_{\text{rgBT}}} + \frac{1}{\sigma_{\text{Overlock}}} + \frac{1}{\sigma_{\text{Tf}}} + \frac{1}{\sigma_{\text{Td}}}$ reaction up to 8 keV neutron energy. Physical Review C, 2013, 87, .	1.1	39
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160	Analyses of shielding benchmark experiments using FENDL-3 cross-section data starter library for ITER and IFMIF applications. <i>Progress in Nuclear Science and Technology</i> , 2014, 4, 322-326.	0.3	8
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384	<p>$\langle \text{mml:math} \text{xmlns:mml="http://www.w3.org/1998/Math/MathML"} \rangle \langle \text{mml:mo} \rangle \langle \text{mml:mi} \rangle n \langle \text{mml:mi} \rangle \langle \text{mml:mo} \rangle \langle \text{mml:mi} \rangle \hat{1}^3 \langle \text{mml:mi} \rangle$</p> <p>$\langle \text{mml:mrow} \rangle \langle \text{mml:mrow} \rangle \langle \text{mml:mmultiscripts} \rangle \langle \text{mml:mrow} \rangle \langle \text{mml:mi} \text{mathvariant="normal"} \rangle U \langle \text{mml:mi} \rangle \langle \text{mml:mrow} \rangle \langle \text{mml:mprescripts} \rangle \langle \text{mml:none} \rangle \langle \text{mml:mrow} \rangle \langle \text{mml:mn} \rangle 235 \langle \text{mml:mn} \rangle \langle \text{mml:mrow} \rangle \langle \text{mml:mmultiscripts} \rangle \langle \text{mml:mrow} \rangle \langle \text{mml:mo} \rangle$</p>		

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405	Supercritical kinetic analysis in simplified system of fuel debris using integral kinetic model. <i>Annals of Nuclear Energy</i> , 2016, 91, 59-64.	0.9	7
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415	Evaluation of neutron nuclear data on xenon isotopes. <i>Journal of Nuclear Science and Technology</i> , 2016, 53, 1310-1320.	0.7	3
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