

# Jack D Law

## List of Publications by Year in descending order

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49  
papers

1,880  
citations

304743

22  
h-index

254184

43  
g-index

53  
all docs

53  
docs citations

53  
times ranked

1096  
citing authors

#	ARTICLE	IF	CITATIONS
1	Countercurrent Actinide Lanthanide Separation Process (ALSEP) Demonstration Test with a Simulated PUREX Raffinate in Centrifugal Contactors on the Laboratory Scale. Applied Sciences (Switzerland), 2020, 10, 7217.	2.5	14
2	Oxidation and extraction of Am(VI) using a monoamidic extractant in 3D printed centrifugal contactors. Journal of Radioanalytical and Nuclear Chemistry, 2018, 318, 35-41.	1.5	5
3	Evaluation of the impacts of gamma radiolysis on an ALSEP process solvent. Journal of Radioanalytical and Nuclear Chemistry, 2018, 316, 855-860.	1.5	6
4	Aqueous Reprocessing of Used Nuclear Fuel. Modern Nuclear Energy Analysis Methods, 2018, , 1-26.	0.1	2
5	An Advanced TALSPEAK Concept for Separating Minor Actinides. Part 1. Process Optimization and Flowsheet Development. Solvent Extraction and Ion Exchange, 2017, 35, 377-395.	2.0	26
6	An Advanced TALSPEAK Concept for Separating Minor Actinides. Part 2. Flowsheet Test with Actinide-spiked Simulant. Solvent Extraction and Ion Exchange, 2017, 35, 396-407.	2.0	25
7	Development and evaluation of a silver mordenite composite sorbent for the partitioning of xenon from krypton in gas compositions. Journal of Nuclear Science and Technology, 2016, 53, 1484-1488.	1.3	6
8	The solvent extraction of Am(VI) using centrifugal contactors. Journal of Radioanalytical and Nuclear Chemistry, 2016, 307, 1833-1836.	1.5	10
9	Microfluidic-Based Sample Chips for Radioactive Solutions. Nuclear Technology, 2015, 189, 301-311.	1.2	2
10	Polyacrylonitrile-Chalcogel Hybrid Sorbents for Radioiodine Capture. Environmental Science & Technology, 2014, 48, 5832-5839.	10.0	90
11	Characterizing Diamylmethylphosphonate (DAAP) as an Americium Ligand for Nuclear Fuel-Cycle Applications. Solvent Extraction and Ion Exchange, 2014, 32, 153-166.	2.0	23
12	Development of a hydrogen mordenite sorbent for the capture of krypton from used nuclear fuel reprocessing off-gas streams. Journal of Nuclear Science and Technology, 2014, 51, 476-481.	1.3	5
13	Microfluidic-Based Robotic Sampling System for Radioactive Solutions. Nuclear Technology, 2014, 185, 216-225.	1.2	0
14	Extraction of Lanthanoids with Diamides of Dipcolinic Acid from Nitric Acid Solutions. II. Synergistic Effect of Ethyl-Tolyl Derivates and Dicarbollide Cobalt. Solvent Extraction and Ion Exchange, 2013, 31, 184-197.	2.0	29
15	Radioactive Iodine and Krypton Control for Nuclear Fuel Reprocessing Facilities. Science and Technology of Nuclear Installations, 2013, 2013, 1-12.	0.8	134
16	Nuclear Fuel nuclear fuel , Reprocessing nuclear fuel reprocessing of. , 2012, , 7142-7156.		0
17	Advanced Remote Maintenance Design for Pilot-Scale Centrifugal Contactors. Nuclear Technology, 2011, 173, 191-199.	1.2	4
18	A Centrifugal Contactor Design to Facilitate Remote Replacement. Nuclear Technology, 2011, 173, 289-299.	1.2	5

#	ARTICLE	IF	CITATIONS
19	The extraction of actinides from nitric acid solutions with diamides of dipicolinic acid. IOP Conference Series: Materials Science and Engineering, 2010, 9, 012068.	0.6	10
20	Selective Extraction of Minor Actinides from Acidic Media Using Symmetric and Asymmetric Dithiophosphinic Acids. Separation Science and Technology, 2010, 45, 1711-1717.	2.5	49
21	The Radiolytic and Thermal Stability of Diamides of Dipicolinic Acid. Separation Science and Technology, 2010, 45, 1706-1710.	2.5	13
22	Pilot-Scale TRUEX Flowsheet Testing for Separation of Actinides and Lanthanides from used Nuclear Fuel. Separation Science and Technology, 2010, 45, 1769-1775.	2.5	15
23	Thirty Stage Annular Centrifugal Contactor Thermal Profile Measurements. Separation Science and Technology, 2010, 45, 310-321.	2.5	6
24	Extraction of uranium(VI) with diamides of dipicolinic acid from nitric acid solutions. Radiochimica Acta, 2009, 97, .	1.2	72
25	Extraction of molybdenum and technetium with diamides of dipicolinic acid from nitric acid solutions. Journal of Radioanalytical and Nuclear Chemistry, 2009, 280, 307-313.	1.5	4
26	Synthesis of symmetric dithiophosphinic acids for "minor actinide" extraction. Inorganica Chimica Acta, 2008, 361, 2522-2532.	2.4	63
27	Extraction of Lanthanides with Diamides of Dipicolinic Acid from Nitric Acid Solutions. I. Separation Science and Technology, 2008, 43, 2606-2618.	2.5	47
28	Reprocessing of spent solvent of the UNEX process. Radiochemistry, 2007, 49, 493-498.	0.7	15
29	Advances in Development of the Fission Product Extraction Process for the Separation of Cesium and Strontium From Spent Nuclear Fuel. , 2007, , .		1
30	Radionuclide Extraction by 2,6-Pyridinedicarboxylamide Derivatives and Chlorinated Cobalt Dicarbolide. Separation Science and Technology, 2006, 41, 2111-2127.	2.5	45
31	Separation of trivalent actinides from lanthanides in an acetate buffer solution using Cyanex 301. Radiochimica Acta, 2006, 94, .	1.2	28
32	Development of Steam Reforming for the Solidification of the Cesium and Strontium Separations Product from Advanced Aqueous Reprocessing of Spent Nuclear Fuel. Separation Science and Technology, 2006, 41, 2147-2162.	2.5	4
33	Radiation stability of regenerated stripping solutions for high-level waste processing. Journal of Radioanalytical and Nuclear Chemistry, 2005, 266, 349-353.	1.5	2
34	Development of a Regenerable Strip Reagent for Treatment of Acidic, Radioactive Waste with Cobalt Dicarbolide-based Solvent Extraction Processes. Solvent Extraction and Ion Exchange, 2005, 23, 59-83.	2.0	11
35	Fission Product Extraction (FPEX): Development of a Novel Solvent for the Simultaneous Separation of Strontium and Cesium from Acidic Solutions. Solvent Extraction and Ion Exchange, 2005, 23, 449-461.	2.0	117
36	Development of a Cobalt Dicarbolide/Polyethylene Glycol Solvent Extraction Process for Separation of Cesium and Strontium to Support Advanced Aqueous Reprocessing. Nuclear Technology, 2004, 147, 284-290.	1.2	27

#	ARTICLE	IF	CITATIONS
37	Development Of The Universal Extraction (Unex) Process For The Simultaneous Recovery Of Cs, Sr, And Actinides From Acidic Radioactive Wastes. Separation Science and Technology, 2003, 38, 2685-2708.	2.5	49
38	UNIVERSAL SOLVENT EXTRACTION (UNEX) FLOWSHEET TESTING FOR THE REMOVAL OF CESIUM, STRONTIUM, AND ACTINIDE ELEMENTS FROM RADIOACTIVE, ACIDIC DISSOLVED CALCINE WASTE. Solvent Extraction and Ion Exchange, 2002, 20, 429-445.	2.0	91
39	Integrated AMP&“PAN, TRUEX, and SREX testing. I. Extended flowsheet testing for separation of surrogate radionuclides from simulated acidic tank waste. Separation Science and Technology, 2002, 37, 1321-1351.	2.5	22
40	Integrated AMP&“PAN, TRUEX, and SREX testing. II. Flowsheet testing for separation of radionuclides from actual acidic radioactive waste. Separation Science and Technology, 2002, 37, 1353-1373.	2.5	22
41	Development and testing of a cobalt dicarbollide based solvent extraction process for the separation of cesium and strontium from acidic tank waste. Separation Science and Technology, 2002, 37, 1807-1831.	2.5	34
42	THE UNIVERSAL SOLVENT EXTRACTION (UNEX) PROCESS. I. DEVELOPMENT OF THE UNEX PROCESS SOLVENT FOR THE SEPARATION OF CESIUM, STRONTIUM, AND THE ACTINIDES FROM ACIDIC RADIOACTIVE WASTE. Solvent Extraction and Ion Exchange, 2001, 19, 1-21.	2.0	322
43	THE UNIVERSAL SOLVENT EXTRACTION (UNEX) PROCESS. II. FLOWSHEET DEVELOPMENT AND DEMONSTRATION OF THE UNEX PROCESS FOR THE SEPARATION OF CESIUM, STRONTIUM, AND ACTINIDES FROM ACTUAL ACIDIC RADIOACTIVE WASTE. Solvent Extraction and Ion Exchange, 2001, 19, 23-36.	2.0	228
44	Development and demonstration of solvent extraction processes for the separation of radionuclides from acidic radioactive waste. Waste Management, 1999, 19, 27-37.	7.4	81
45	Development of a universal solvent for the decontamination of acidic liquid radioactive wastes. European Physical Journal D, 1999, 49, 931-936.	0.4	8
46	Demonstration of the TRUEX Process for the Treatment of Actual High-Activity Tank Waste at the INEEL Using Centrifugal Contractors. , 1998, , 245-253.		2
47	Development and Testing of Srex Flowsheets for Treatment of Idaho Chemical Processing Plant Sodium-Bearing Waste Using Centrifugal Contactors. Separation Science and Technology, 1997, 32, 223-240.	2.5	36
48	EXTRACTION OF LEAD AND STRONTIUM FROM HAZARDOUS WASTE STREAMS BY SOLVENT EXTRACTION WITH 4',4',(5')-DI-(T-BUTYLDICYCLOHEXO)-18-CROWN-6. Solvent Extraction and Ion Exchange, 1997, 15, 65-78.	2.0	13
49	Evaluation of the SREX Solvent Extraction Process for the Removal Of <sup>90</sup> Sr and Hazardous Metals from Acidic Nuclear Waste Solutions Containing High Concentrations of Interfering Alkali Metal Ions. Separation Science and Technology, 1997, 32, 241-253.	2.5	47