Thomas J Downar

List of Publications by Year in descending order

Source: https://exaly.com/author-pdf/9586257/publications.pdf

Version: 2024-02-01

26 432 11
papers citations h-index

27 27 27 224 all docs docs citations times ranked citing authors

752698

20

g-index

| # | Article | IF | CITATIONS |
|----|---|-----|-----------|
| 1 | Stability and accuracy of 3D neutron transport simulations using the 2D/1D method in MPACT. Journal of Computational Physics, 2016, 326, 612-628. | 3.8 | 94 |
| 2 | An optimally diffusive Coarse Mesh Finite Difference method to accelerate neutron transport calculations. Annals of Nuclear Energy, 2016, 95, 116-124. | 1.8 | 72 |
| 3 | VERA Core Simulator Methodology for Pressurized Water Reactor Cycle Depletion. Nuclear Science and Engineering, 2017, 185, 217-231. | 1.1 | 66 |
| 4 | Transient analysis of C5G7-TD benchmark with MPACT. Annals of Nuclear Energy, 2019, 125, 107-120. | 1.8 | 25 |
| 5 | Implementation of Two-Level Coarse-Mesh Finite Difference Acceleration in an Arbitrary Geometry, Two-Dimensional Discrete Ordinates Transport Method. Nuclear Science and Engineering, 2008, 158, 289-298. | 1.1 | 22 |
| 6 | Cell Homogenization Method for Pin-by-Pin Neutron Transport Calculations. Nuclear Science and Engineering, 2011, 169, 1-18. | 1.1 | 20 |
| 7 | A Multilevel Quasi-Static Kinetics Method for Pin-Resolved Transport Transient Reactor Analysis. Nuclear Science and Engineering, 2016, 182, 435-451. | 1.1 | 17 |
| 8 | Analysis of the Core Power Response during a PWR Rod Ejection Transient Using the PARCS Nodal Code and the DeCART MOC Code. Nuclear Science and Engineering, 2012, 170, 151-167. | 1.1 | 16 |
| 9 | A 2-D/1-D Transverse Leakage Approximation Based on Azimuthal, Fourier Moments. Nuclear Science and Engineering, 2017, 185, 243-262. | 1.1 | 15 |
| 10 | Subplane collision probabilities method applied to control rod cusping in 2D/1D. Annals of Nuclear Energy, 2018, 118, 1-14. | 1.8 | 13 |
| 11 | Theoretical Convergence Rate Lower Bounds for Variants of Coarse Mesh Finite Difference to Accelerate Neutron Transport Calculations. Nuclear Science and Engineering, 2017, 186, 224-238. | 1.1 | 12 |
| 12 | Impact of improved neutronic methodology on the cladding response during a PWR reactivity initiated accident. Nuclear Engineering and Design, 2013, 262, 180-188. | 1.7 | 11 |
| 13 | Validation of the U.S. NRC Coupled Code System TRITON/TRACE/PARCS Using the Special Power Excursion Reactor Test III. Nuclear Technology, 2013, 183, 504-514. | 1.2 | 9 |
| 14 | Parameter Sensitivity Study of Boiling and Two-Phase Flow Models in CFD. Journal of Computational Multiphase Flows, 2012, 4, 411-425. | 0.8 | 8 |
| 15 | Fourier Convergence Analysis of the Infinite Homogenous Multigroup Time-Dependent Boltzmann Transport Equation Using Discrete Ordinates Formulation. Nuclear Science and Engineering, 2017, 186, 23-36. | 1.1 | 5 |
| 16 | Improved Accuracy in the 2-D/1-D Method with Anisotropic Transverse Leakage and Cross-Section Homogenization. Nuclear Science and Engineering, 2018, 192, 219-239. | 1.1 | 4 |
| 17 | Quasi-Diffusion Method with 3-D Cross Sections for TREAT Core Analysis. Nuclear Technology, 2020, 206, 825-838. | 1.2 | 4 |
| 18 | Validation of Pin-Resolved Reaction Rates, Kinetics Parameters, and Linear Source MOC in MPACT. Nuclear Science and Engineering, 2021, 195, 50-68. | 1.1 | 4 |

| # | Article | IF | CITATIONS |
|----|---|-----|-----------|
| 19 | Transient Multilevel Scheme with One-Group CMFD Acceleration. Nuclear Science and Engineering, 2021, 195, 741-765. | 1.1 | 4 |
| 20 | The Subray Method of Characteristics. Nuclear Science and Engineering, 2019, 193, 601-621. | 1.1 | 3 |
| 21 | The Legendre Polynomial Axial Expansion Method. Nuclear Science and Engineering, 2023, 197, 291-307. | 1.1 | 3 |
| 22 | Sensitivity and Uncertainty of OECD Benchmark Ringhals-1 TRACE/PARCS Stability Prediction. Nuclear Technology, 2012, 180, 383-398. | 1.2 | 2 |
| 23 | Polar Parity for Efficient Evaluation of Anisotropic Transverse Leakage in the 2D/1D Transport Method. Nuclear Science and Engineering, 2019, 193, 1291-1309. | 1.1 | 1 |
| 24 | Enhanced Lasso Regularization-Based Self-Adaptive Feature Selection Algorithm for the High-Dimensional Uncertainty Quantification of TREAT Transient Test Modeling. Nuclear Technology, 2020, 206, 839-861. | 1.2 | 1 |
| 25 | Development and Analysis of TREAT Neutronics Benchmarks. Nuclear Technology, 2020, 206, 805-824. | 1.2 | 1 |
| 26 | SP3 Limit of the 2D/1D Transport Equations with Varying Degrees of Angular Coupling. Journal of Computational and Theoretical Transport, 2020, 49, 303-330. | 0.8 | 0 |