

# Marek Rubel

## List of Publications by Year in descending order

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papers

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docs citations

202  
times ranked

2094  
citing authors

#	ARTICLE	IF	CITATIONS
1	Plasmaâ€‘surface interaction in the stellarator W7-X: conclusions drawn from operation with graphite plasma-facing components. Nuclear Fusion, 2022, 62, 016006.	1.6	12
2	Application of Ion Beam Analysis in Studies of First Wall Materials in Controlled Fusion Devices. Physics, 2022, 4, 37-50.	0.5	0
3	An overview of tritium retention in dust particles from the JET-ILW divertor. Physica Scripta, 2022, 97, 024008.	1.2	3
4	Isotope removal experiment in JET-ILW in view of T-removal after the 2nd DT campaign at JET. Physica Scripta, 2022, 97, 044001.	1.2	7
5	Studies on the behaviour of titanium activation foils during long-term exposure at the JET tokamak. Fusion Engineering and Design, 2022, 177, 113056.	1.0	1
6	3D deposition patterns of deuterium retention and impurities in the COMPASS divertor: a data-driven root cause analysis and prediction approach. Fusion Engineering and Design, 2022, 179, 113118.	1.0	1
7	The upgraded TOMAS device: A toroidal plasma facility for wall conditioning, plasma production, and plasmaâ€‘surface interaction studies. Review of Scientific Instruments, 2021, 92, 023506.	0.6	13
8	Global distribution of tritium in JET with the ITER-like wall. Nuclear Materials and Energy, 2021, 26, 100930.	0.6	7
9	First mirror erosionâ€‘deposition studies in JET using an ITER-like mirror test assembly. Nuclear Fusion, 2021, 61, 046022.	1.6	13
10	Data on erosion and hydrogen fuel retention in Beryllium plasma-facing materials. Nuclear Materials and Energy, 2021, 27, 100994.	0.6	21
11	Characterization of neutral particle fluxes from ICWC and ECWC plasmas in the TOMAS facility. Physica Scripta, 2021, 96, 124025.	1.2	7
12	Fuel retention and erosion-deposition on inner wall cladding tiles in JET-ILW. Physica Scripta, 2021, 96, 124071.	1.2	7
13	Evaluation of tritium retention in plasma facing components during JET tritium operations. Physica Scripta, 2021, 96, 124075.	1.2	14
14	Determination of retained tritium from ILW dust particles in JET. Nuclear Materials and Energy, 2020, 22, 100673.	0.6	7
15	Ion beam analysis of fusion plasma-facing materials and components: facilities and research challenges. Nuclear Fusion, 2020, 60, 025001.	1.6	54
16	Tritium distribution analysis of Be limiter tiles from JET-ITER like wall campaigns using imaging plate technique and $\text{I}^2$ -ray induced X-ray spectrometry. Fusion Engineering and Design, 2020, 160, 111959.	1.0	6
17	Search for mobilised dust during operations with equipment for remote handling in JET with ITER-like wall. Physica Scripta, 2020, T171, 014048.	1.2	8
18	Comparison of Hydrogen Isotope Retention in Divertor Tiles of JET with the ITER-Like Wall Following Campaigns in 2011â€‘2012 and 2015â€‘2016. Fusion Science and Technology, 2020, 76, 439-445.	0.6	3

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19	Metallography and mechanical parameters of plasma-exposed plasma-facing materials and components. Physica Scripta, 2020, T171, 014042.	1.2	5
20	Deposition in the tungsten divertor during the 2011â€“2016 campaigns in JET with ITER-like wall. Physica Scripta, 2020, T171, 014044.	1.2	11
21	Surface morphology of the bulk tungsten divertor tiles from JET ITER-like wall. Physica Scripta, 2020, T171, 014010.	1.2	4
22	Fuel inventory and impurity deposition in castellated tungsten tiles in KSTAR: experiment and modelling. Physica Scripta, 2020, T171, 014049.	1.2	2
23	Decommissioning of TEXTOR: properties of the Inconel liner. Physica Scripta, 2020, T171, 014036.	1.2	0
24	Micro-analyses of dust particles generated in the JET tokamak with the ITER-like wall. Nuclear Fusion, 2020, 60, 126031.	1.6	9
25	Erosion, screening, and migration of tungsten in the JET divertor. Nuclear Fusion, 2019, 59, 096035.	1.6	60
26	Implementation and exploitation of JET enhancements at different fuel mixtures in preparation for DT operation and next step devices. Fusion Engineering and Design, 2019, 146, 741-744.	1.0	2
27	Beryllium melting and erosion on the upper dump plates in JET during three ITER-like wall campaigns. Nuclear Fusion, 2019, 59, 086009.	1.6	45
28	First mirror test in JET for ITER: Complete overview after three ILW campaigns. Nuclear Materials and Energy, 2019, 19, 59-66.	0.6	24
29	Diagnostic mirrors for ITER: research in the frame of International Tokamak Physics Activity. Nuclear Fusion, 2019, 59, 066029.	1.6	41
30	Fusion Neutrons: Tritium Breeding and Impact on Wall Materials and Components of Diagnostic Systems. Journal of Fusion Energy, 2019, 38, 315-329.	0.5	41
31	Analysis of deposited layers with deuterium and impurity elements on samples from the divertor of JET with ITER-like wall. Journal of Nuclear Materials, 2019, 516, 202-213.	1.3	18
32	Material migration and fuel retention studies during the JET carbon divertor campaigns. Fusion Engineering and Design, 2019, 138, 78-108.	1.0	25
33	Ion Microbeam Analyses of Dust Particles and Codeposits from JET with the ITER-Like Wall. Analytical Chemistry, 2018, 90, 5744-5752.	3.2	12
34	Plasma-wall interaction on the divertor tiles of JET ITER-like wall from the viewpoint of micro/nanosopic observations. Fusion Engineering and Design, 2018, 136, 199-204.	1.0	5
35	Dust generation in tokamaks: Overview of beryllium and tungsten dust characterisation in JET with the ITER-like wall. Fusion Engineering and Design, 2018, 136, 579-586.	1.0	52
36	Tritium retention characteristics in dust particles in JET with ITER-like wall. Nuclear Materials and Energy, 2018, 17, 279-283.	0.6	20

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37	Review on global migration, fuel retention and modelling after TEXTOR decommission. Nuclear Materials and Energy, 2018, 17, 83-112.	0.6	9
38	Correlation of surface chemical states with hydrogen isotope retention in divertor tiles of JET with ITER-Like Wall. Fusion Engineering and Design, 2018, 132, 24-28.	1.0	15
39	Modelling of deposition and erosion of injected WF6 and MoF6 in TEXTOR. Nuclear Materials and Energy, 2017, 12, 564-568.	0.6	4
40	Assessment of erosion, deposition and fuel retention in the JET-ILW divertor from ion beam analysis data. Nuclear Materials and Energy, 2017, 12, 559-563.	0.6	28
41	Whole-machine material migration studies in the TEXTOR tokamak with molybdenum. Nuclear Materials and Energy, 2017, 12, 518-523.	0.6	6
42	Engineering design and analysis of an ITER-like first mirror test assembly on JET. Fusion Engineering and Design, 2017, 123, 1054-1057.	1.0	5
43	Studies of dust from JET with the ITER-Like Wall: Composition and internal structure. Nuclear Materials and Energy, 2017, 12, 582-587.	0.6	41
44	Plasma impact on diagnostic mirrors in JET. Nuclear Materials and Energy, 2017, 12, 506-512.	0.6	25
45	Design of an ICRF system for plasma-wall interactions and RF plasma production studies on TOMAS. Fusion Engineering and Design, 2017, 123, 317-320.	1.0	4
46	Micro-/nano-characterization of the surface structures on the divertor tiles from JET ITER-like wall. Fusion Engineering and Design, 2017, 116, 1-4.	1.0	14
47	Overview of the JET ITER-like wall divertor. Nuclear Materials and Energy, 2017, 12, 499-505.	0.6	46
48	Be ITER-like wall at the JET tokamak under plasma. Physica Scripta, 2017, T170, 014049.	1.2	4
49	Fuel inventory and deposition in castellated structures in JET-ILW. Nuclear Fusion, 2017, 57, 066027.	1.6	25
50	Surface composition and structure of divertor tiles following the JET tokamak operation with the ITER-like wall. Nuclear Fusion, 2017, 57, 076027.	1.6	13
51	Investigation of probe surfaces after ion cyclotron wall conditioning in ASDEX upgrade. Nuclear Materials and Energy, 2017, 12, 733-735.	0.6	3
52	Overview of wall probes for erosion and deposition studies in the TEXTOR tokamak. Matter and Radiation at Extremes, 2017, 2, 87-104.	1.5	23
53	Overview of fuel inventory in JET with the ITER-like wall. Nuclear Fusion, 2017, 57, 086045.	1.6	47
54	Impurity re-distribution in the corner regions of the JET divertor. Physica Scripta, 2017, T170, 014060.	1.2	6

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55	Ageing of structural materials in tokamaks: TEXTOR liner study. Physica Scripta, 2017, T170, 014053.	1.2	3
56	Fine metal dust particles on the wall probes from JET-ILW. Physica Scripta, 2017, T170, 014038.	1.2	22
57	16th International Conference on Plasma-Facing Materials and Components for Fusion Applications. Physica Scripta, 2017, T170, 010201.	1.2	0
58	Metallic mirrors for plasma diagnosis in current and future reactors: tests for ITER and DEMO. Physica Scripta, 2017, T170, 014061.	1.2	12
59	Analyses of microstructure, composition and retention of hydrogen isotopes in divertor tiles of JET with the ITER-like wall. Physica Scripta, 2017, T170, 014031.	1.2	13
60	Time-resolved deposition in the remote region of the JET-ILW divertor: measurements and modelling. Physica Scripta, 2017, T170, 014059.	1.2	6
61	Plasma-wall interaction studies within the EUROfusion consortium: progress on plasma-facing components development and qualification. Nuclear Fusion, 2017, 57, 116041.	1.6	75
62	A combined segmented anode gas ionization chamber and time-of-flight detector for heavy ion elastic recoil detection analysis. Review of Scientific Instruments, 2016, 87, 103303.	0.6	98
63	Impact of helium implantation and ion-induced damage on reflectivity of molybdenum mirrors. Nuclear Instruments & Methods in Physics Research B, 2016, 382, 91-95.	0.6	15
64	Experience of handling beryllium, tritium and activated components from JET ITER like wall. Physica Scripta, 2016, T167, 014057.	1.2	18
65	Ion beam analysis of tungsten layers in EUROFER model systems and carbon plasma facing components. Nuclear Instruments & Methods in Physics Research B, 2016, 371, 355-359.	0.6	11
66	The role and application of ion beam analysis for studies of plasma-facing components in controlled fusion devices. Nuclear Instruments & Methods in Physics Research B, 2016, 371, 4-11.	0.6	18
67	Local migration studies of high-Zmetals in the TEXTOR tokamak. Physica Scripta, 2016, T167, 014058.	1.2	9
68	Dust survey following the final shutdown of TEXTOR: metal particles and fuel retention. Physica Scripta, 2016, T167, 014059.	1.2	10
69	Plasma cleaning of beryllium coated mirrors. Physica Scripta, 2016, T167, 014069.	1.2	24
70	Nitrogen removal from plasma-facing components by ion cyclotron wall conditioning in TEXTOR. Journal of Nuclear Materials, 2015, 463, 688-692.	1.3	9
71	Tracer techniques for the assessment of material migration and surface modification of plasma-facing components. Journal of Nuclear Materials, 2015, 463, 280-284.	1.3	13
72	First dust study in JET with the ITER-like wall: sampling, analysis and classification. Nuclear Fusion, 2015, 55, 113033.	1.6	51

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73	Characterisation of surface layers formed on plasma-facing components in controlled fusion devices: Role of heavy ion elastic recoil detection. <i>Vacuum</i> , 2015, 122, 260-267.	1.6	11
74	Material deposition on inner divertor quartz-micro balances during ITER-like wall operation in JET. <i>Journal of Nuclear Materials</i> , 2015, 463, 796-799.	1.3	8
75	Beryllium migration in JET ITER-like wall plasmas. <i>Nuclear Fusion</i> , 2015, 55, 063021.	1.6	83
76	Co-deposited layers in the divertor region of JET-ILW. <i>Journal of Nuclear Materials</i> , 2015, 463, 814-817.	1.3	32
77	Global erosion and deposition patterns in JET with the ITER-like wall. <i>Journal of Nuclear Materials</i> , 2015, 463, 157-161.	1.3	48
78	An overview of the comprehensive First Mirror Test in JET with ITER-like wall. <i>Physica Scripta</i> , 2014, T159, 014011.	1.2	59
79	Overview of nitrogen-15 application as a tracer gas for material migration and retention studies in tokamaks. <i>Physica Scripta</i> , 2014, T159, 014042.	1.2	12
80	Material migration patterns and overview of first surface analysis of the JET ITER-like wall. <i>Physica Scripta</i> , 2014, T159, 014010.	1.2	75
81	Post-mortem measurements of fuel retention at JET. <i>Physica Scripta</i> , 2014, T159, 014052.	1.2	6
82	First results and surface analysis strategy for plasma-facing components after JET operation with the ITER-like wall. <i>Physica Scripta</i> , 2014, T159, 014016.	1.2	30
83	Impact of ion cyclotron wall conditioning on fuel removal from plasma-facing components at TEXTOR. <i>Physica Scripta</i> , 2014, T159, 014017.	1.2	9
84	Laser-assisted cleaning of beryllium-containing mirror samples from JET and PISCES-B. <i>Fusion Engineering and Design</i> , 2014, 89, 122-130.	1.0	23
85	The JET technology program in support of ITER. <i>Fusion Engineering and Design</i> , 2014, 89, 896-900.	1.0	14
86	Performances of Rh and Mo mirrors under JET exposure. <i>Journal of Nuclear Materials</i> , 2013, 438, S1187-S1191.	1.3	14
87	Self-consistent application of ion cyclotron wall conditioning for co-deposited layer removal and recovery of tokamak operation on TEXTOR. <i>Nuclear Fusion</i> , 2013, 53, 123001.	1.6	15
88	Tungsten migration studies by controlled injection of volatile compounds. <i>Journal of Nuclear Materials</i> , 2013, 438, S170-S174.	1.3	17
89	Injection of nitrogen-15 tracer into ASDEX-Upgrade: New technique in material migration studies. <i>Journal of Nuclear Materials</i> , 2013, 438, S616-S619.	1.3	19
90	Assessment of cleaning methods for first mirrors tested in JET for ITER. <i>Journal of Nuclear Materials</i> , 2013, 438, S1241-S1244.	1.3	6

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91	Overview of erosion-deposition diagnostic tools for the ITER-Like Wall in the JET tokamak. Journal of Nuclear Materials, 2013, 438, S1204-S1207.	1.3	44
92	Deuterium inventory in Tore Supra: Coupled carbon-deuterium balance. Journal of Nuclear Materials, 2013, 438, S120-S125.	1.3	38
93	Development of laser-based techniques for <i>in situ</i> characterization of the first wall in ITER and future fusion devices. Nuclear Fusion, 2013, 53, 093002.	1.6	99
94	Efficiency of fuel removal techniques tested on plasma-facing components from the TEXTOR tokamak. Fusion Engineering and Design, 2012, 87, 935-940.	1.0	7
95	Nuclear reaction and heavy ion ERD analysis of wall materials from controlled fusion devices: Deuterium and nitrogen-15 studies. Nuclear Instruments & Methods in Physics Research B, 2012, 273, 113-117.	0.6	20
96	JET ITER-like wall overview and experimental programme. Physica Scripta, 2011, T145, 014001.	1.2	263
97	Theoretical and experimental studies on molybdenum and stainless steel mirrors cleaning by high repetition rate laser beam. Fusion Engineering and Design, 2011, 86, 1728-1731.	1.0	23
98	Overview of JET post-mortem results following the 2007-9 operational period, and comparisons with previous campaigns. Physica Scripta, 2011, T145, 014003.	1.2	20
99	Deposition of <sup>13</sup> C tracer in the JET MkII-HD divertor. Physica Scripta, 2011, T145, 014004.	1.2	15
100	Nitrogen and neon retention in plasma-facing materials. Journal of Nuclear Materials, 2011, 415, S223-S226.	1.3	34
101	Overview of experimental preparation for the ITER-Like Wall at JET. Journal of Nuclear Materials, 2011, 415, S936-S942.	1.3	29
102	An overview of nuclear micro-beam analysis of surface and bulk fuel retention in carbon-fibre composites from Tore Supra. Journal of Nuclear Materials, 2011, 415, S761-S764.	1.3	8
103	Removal of beryllium-containing films deposited in JET from mirror surfaces by laser cleaning. Journal of Nuclear Materials, 2011, 415, S1199-S1202.	1.3	30
104	Overview of material migration and mixing, fuel retention and cleaning of ITER-like castellated structures in TEXTOR. Journal of Nuclear Materials, 2011, 415, S289-S292.	1.3	20
105	Laser-based and thermal methods for fuel removal and cleaning of plasma-facing components. Journal of Nuclear Materials, 2011, 415, S801-S804.	1.3	18
106	Multi machine scaling of fuel retention in 4 carbon dominated tokamaks. Journal of Nuclear Materials, 2011, 415, S735-S739.	1.3	20
107	Overview of the second stage in the comprehensive mirrors test in JET. Physica Scripta, 2011, T145, 014070.	1.2	20
108	Micro-distribution of fuel and metal in carbon-based plasma-facing materials. Physica Scripta, 2011, T145, 014014.	1.2	8

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109	Structure Materials in Fusion Reactors: Issues Related to Tritium, Radioactivity and Radiation-Induced Effects. <i>Fusion Science and Technology</i> , 2010, 57, 474-482.	0.6	2
110	First Mirrors Test in JET for ITER: An overview of optical performance and surface morphology. <i>Nuclear Instruments and Methods in Physics Research, Section A: Accelerators, Spectrometers, Detectors and Associated Equipment</i> , 2010, 623, 818-822.	0.7	15
111	Nuclear micro-beam analysis of deuterium distribution in carbon fibre composites for controlled fusion devices. <i>Nuclear Instruments &amp; Methods in Physics Research B</i> , 2010, 268, 1833-1837.	0.6	15
112	Survey of dust formed in the TEXTOR tokamak: structure and fuel retention. <i>Physica Scripta</i> , 2009, T138, 014025.	1.2	26
113	Current status of the JET ITER-like Wall Project. <i>Physica Scripta</i> , 2009, T138, 014030.	1.2	42
114	Material mixing on plasma-facing components: Compound formation. <i>Journal of Nuclear Materials</i> , 2009, 386-388, 740-743.	1.3	16
115	Analysis of fuel retention in plasma-facing components from controlled fusion devices. <i>Nuclear Instruments &amp; Methods in Physics Research B</i> , 2009, 267, 711-717.	0.6	18
116	In-situ measurements of carbon and deuterium deposition using the fast reciprocating probe in TEXTOR. <i>Journal of Nuclear Materials</i> , 2009, 390-391, 179-182.	1.3	3
117	Post-mortem measurements of fuel retention at JET with MKII-SRP divertor. <i>Journal of Nuclear Materials</i> , 2009, 390-391, 631-634.	1.3	17
118	Nitrogen-assisted removal of deuterated carbon layers. <i>Journal of Nuclear Materials</i> , 2009, 390-391, 647-650.	1.3	11
119	An overview of a comprehensive First Mirror Test for ITER at JET. <i>Journal of Nuclear Materials</i> , 2009, 390-391, 1066-1069.	1.3	40
120	An overview of erosion&quot;deposition studies for the JET Mk II high delta divertor. <i>Physica Scripta</i> , 2009, T138, 014005.	1.2	18
121	Deposition results from rotating collector diagnostics in JET. <i>Physica Scripta</i> , 2009, T138, 014023.	1.2	13
122	Thermal load testing of erosion-monitoring beryllium marker tile for the ITER-Like Wall Project at JET. <i>Fusion Engineering and Design</i> , 2008, 83, 1072-1076.	1.0	11
123	Modelling of carbon migration during JET13C injection experiments. <i>Nuclear Fusion</i> , 2008, 48, 105002.	1.6	29
124	Beryllium coatings on metals for marker tiles at JET: development of process and characterization of layers. <i>Physica Scripta</i> , 2007, T128, 157-161.	1.2	63
125	Overview of the ITER-like wall project. <i>Physica Scripta</i> , 2007, T128, 137-143.	1.2	183
126	Tungsten and beryllium armour development for the JET ITER-like wall project. <i>Nuclear Fusion</i> , 2007, 47, 222-227.	1.6	32

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127	Characterization and heat flux testing of beryllium coatings on Inconel for JET ITER-like wall project. Physica Scripta, 2007, T128, 166-170.	1.2	23
128	Gas balance and fuel retention in fusion devices. Nuclear Fusion, 2007, 47, 1112-1120.	1.6	94
129	Experience with bulk tungsten test-limiters under high heat loads: melting and melt layer propagation. Physica Scripta, 2007, T128, 81-86.	1.2	51
130	R&D on full tungsten divertor and beryllium wall for JET ITER-like wall project. Fusion Engineering and Design, 2007, 82, 1839-1845.	1.0	66
131	Treatment of ITER plasma facing components: Current status and remaining open issues before ITER implementation. Fusion Engineering and Design, 2007, 82, 2390-2398.	1.0	28
132	Efficacy of photon cleaning of JET divertor tiles. Journal of Nuclear Materials, 2007, 363-365, 341-345.	1.3	25
133	Erosion of a tungsten limiter under high heat flux in TEXTOR. Journal of Nuclear Materials, 2007, 363-365, 96-100.	1.3	38
134	Erosion and deposition in the JET MkII-SRP divertor. Journal of Nuclear Materials, 2007, 363-365, 287-293.	1.3	32
135	Structural studies of deposited layers on JET MkII-SRP inner divertor tiles. Journal of Nuclear Materials, 2007, 363-365, 190-195.	1.3	34
136	Diagnostic mirrors for ITER: A material choice and the impact of erosion and deposition on their performance. Journal of Nuclear Materials, 2007, 363-365, 1395-1402.	1.3	94
137	Tritium retention in next step devices and the requirements for mitigation and removal techniques. Plasma Physics and Controlled Fusion, 2006, 48, B189-B199.	0.9	83
138	Reactor Aspects of Fusion: Issues Related to Materials, Radioactivity and Radiation-Induced Effects. Fusion Science and Technology, 2006, 49, 465-473.	0.6	4
139	Overview of material re-deposition and fuel retention studies at JET with the Gas Box divertor. Nuclear Fusion, 2006, 46, 350-366.	1.6	89
140	Analysis of plasma facing materials: material migration and fuel retention. Physica Scripta, 2006, T123, 54-65.	1.2	10
141	Exposure of metal mirrors in the scrape-off layer of TEXTOR. Journal of Nuclear Materials, 2005, 337-339, 1116-1120.	1.3	32
142	<sup>13</sup> C transport studies in L-mode divertor plasmas on DIII-D. Journal of Nuclear Materials, 2005, 337-339, 30-34.	1.3	38
143	Accelerator-based ion beam analysis of fusion reactor materials. Vacuum, 2005, 78, 255-261.	1.6	20
144	Diagnostics for studying deposition and erosion processes in JET. Fusion Engineering and Design, 2005, 74, 745-749.	1.0	21

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145	Material erosion and migration in tokamaks. Plasma Physics and Controlled Fusion, 2005, 47, B303-B322.	0.9	105
146	Modeling of erosion and deposition patterns on W and Ta twin limiters exposed to the TEXTOR edge plasmas. Journal of Nuclear Materials, 2004, 329-333, 732-736.	1.3	8
147	Beryllium and carbon films in JET following D-T operation. Journal of Nuclear Materials, 2003, 313-316, 321-326.	1.3	56
148	Erosion/deposition in JET during the period 1999-2001. Journal of Nuclear Materials, 2003, 313-316, 419-423.	1.3	69
149	B4C-limiter experiments at TEXTOR. Journal of Nuclear Materials, 2003, 313-316, 223-229.	1.3	8
150	Ion beam analysis methods in the studies of plasma facing materials in controlled fusion devices. Vacuum, 2003, 70, 423-428.	1.6	19
151	Performance and erosion of a tungsten brush limiter exposed at the TEXTOR tokamak. Journal of Nuclear Materials, 2003, 313-316, 67-71.	1.3	18
152	Short and long range transport of materials eroded from wall components in fusion devices. Journal of Nuclear Materials, 2003, 313-316, 311-320.	1.3	49
153	Thick Co-Deposits and Dust in Controlled Fusion Devices with Carbon Walls: Fuel Inventory and Growth Rate of Co-Deposited Layers. Physica Scripta, 2003, T103, 20.	1.2	41
154	Testing of Tungsten and Tantalum Limiters at the TEXTOR Tokamak: Material Performance and Deuterium Retention. Physica Scripta, 2003, T103, 59.	1.2	21
155	Retention of neon in graphite after ion beam implantation or exposures to the scrape-off layer plasma in the TEXTOR tokamak. Journal of Vacuum Science and Technology A: Vacuum, Surfaces and Films, 2002, 20, 138-145.	0.9	7
156	Tritium and Deuterium Retention in Graphite Limiters in TEXTOR. Fusion Science and Technology, 2002, 41, 924-928.	0.6	10
157	Overview of fuel retention in composite and tungsten limiters. Journal of Nuclear Materials, 2002, 307-311, 111-115.	1.3	19
158	TEM and EELS characterization of carbon dust and co-deposited layers from the TEXTOR tokamak. Journal of Nuclear Materials, 2002, 307-311, 1289-1293.	1.3	11
159	Erosion and redeposition of wall material in controlled fusion devices. Vacuum, 2002, 67, 399-408.	1.6	56
160	Dust particles in controlled fusion devices: morphology, observations in the plasma and influence on the plasma performance. Nuclear Fusion, 2001, 41, 1087-1099.	1.6	96
161	In-Situ Measurement of Trapped Hydrogen by Laser Desorption in TEXTOR-94. Physica Scripta, 2001, T94, 102.	1.2	13
162	Carbon Particles Emission, Brittle Destruction and Co-deposit Formation: Experience from Electron Beam Experiments and Controlled Fusion Devices. Physica Scripta, 2001, T91, 36.	1.2	16

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163	Hydrogen inventories in nuclear fusion devices. Journal of Nuclear Materials, 2001, 290-293, 381-388.	1.3	95
164	Investigation of carbon transport in the scrape-off layer of TEXTOR-94. Journal of Nuclear Materials, 2001, 290-293, 362-366.	1.3	55
165	Fuel accumulation in co-deposited layers on plasma facing components. Journal of Nuclear Materials, 2001, 290-293, 473-477.	1.3	54
166	Comparison of impurity production, recycling and power deposition on carbon and tungsten limiters in TEXTOR-94. Journal of Nuclear Materials, 2001, 290-293, 276-280.	1.3	15
167	Simulation calculations of mutual contamination between tungsten and carbon and its impact on plasma surface interactions. Journal of Nuclear Materials, 2001, 290-293, 303-307.	1.3	16
168	Erosion and deposition effects on the vessel wall of TEXTOR-94. Journal of Nuclear Materials, 2001, 290-293, 341-345.	1.3	3
169	Operation of TEXTOR-94 with tungsten poloidal main limiters. Journal of Nuclear Materials, 2001, 290-293, 947-952.	1.3	42
170	Wall conditioning by microwave generated plasmas in a toroidal magnetic field. Journal of Nuclear Materials, 2001, 290-293, 1180-1184.	1.3	10
171	Application of tungsten for plasma limiters in TEXTOR. Journal of Nuclear Materials, 2000, 283-287, 1128-1133.	1.3	42
172	Simulation study of carbon and tungsten deposition on W/C twin test limiter in TEXTOR-94. Journal of Nuclear Materials, 2000, 283-287, 1182-1186.	1.3	4
173	Graphite-tungsten twin limiters in studies of material mixing processes on high heat flux components. Journal of Nuclear Materials, 2000, 283-287, 1089-1093.	1.3	14
174	Molybdenum limiters for Extrap-T2 upgrade: surface properties and high heat flux testing. Fusion Engineering and Design, 2000, 49-50, 323-329.	1.0	3
175	Material mixing on W/C twin limiter in TEXTOR-94. Fusion Engineering and Design, 2000, 49-50, 355-362.	1.0	15
176	Comparison of tokamak behaviour with tungsten and low-Z plasma facing materials. Plasma Physics and Controlled Fusion, 2000, 42, B293-B310.	0.9	48
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