Timur Kulsartov

List of Publications by Year
in descending order

56	527	623734	794594
papers	citations	14 h-index	19 g-index

56	56	56	233
all docs	docs citations	times ranked	citing authors

#	Article	IF	Citations
1	Investigation of hydrogen and deuterium impact on the release of tritium from two-phase lithium ceramics under reactor irradiation. Nuclear Materials and Energy, 2022, 30, 101115.	1.3	5
2	Studies of two-phase lithium ceramics Li4SiO4-Li2TiO3 under conditions of neutron irradiation. Nuclear Materials and Energy, 2022, 30, 101129.	1.3	6
3	Analysis of the reactor experiments results on the study of gas evolution from two-phase Li2TiO3-Li4SiO4 lithium ceramics. Nuclear Materials and Energy, 2022, 30, 101132.	1.3	5
4	Experiments on tritium generation and yield from lithium ceramics during neutron irradiation. International Journal of Hydrogen Energy, 2021, 46, 9186-9192.	7.1	19
5	The study of deuterium permeability of film-forming inhibitors with the addition of fullerenes. International Journal of Hydrogen Energy, 2021, 46, 7426-7431.	7.1	2
6	Radiation resistance of single-mode optical fibres with view to in-reactor applications. Nuclear Materials and Energy, 2021, 27, 100981.	1.3	16
7	Modeling of hydrogen isotopes release from lithium ceramics Li2TiO3 during in-situ experiments using vacuum extraction method. Fusion Engineering and Design, 2021, 170, 112705.	1.9	13
8	Determination of the activation energy of tritium diffusion in ceramic breeders by reactor power variation. Fusion Engineering and Design, 2021, 172, 112783.	1.9	10
9	Features of the in-situ experiments on studying of tritium release from lithium ceramic Li2TiO3 using vacuum extraction method. Fusion Engineering and Design, 2021, 172, 112703.	1.9	13
10	Radiation Resistance of Single-Mode Optical Fibers at Î» = 1.55 Î¼m Under Irradiation at IVG.1M Nuclear Reactor. IEEE Transactions on Nuclear Science, 2020, 67, 2162-2171.	2.0	11
11	Reactor studies of tritium release from lead-lithium eutectic Li15.7Pb with deuterium over the sample. Nuclear Materials and Energy, 2020, 25, 100868.	1.3	2
12	Analysis of the Reactor Experiment Results on Irradiation of Lead-Lithium Eutectic. Fusion Science and Technology, 2020, 76, 632-641.	1.1	3
13	In Situ Determination of Parameters of Hydrogen Isotopes Interaction with Materials Using Dynamic Sorption Method. Fusion Science and Technology, 2020, 76, 333-340.	1.1	2
14	Reactor studies of hydrogen isotopes interaction with lithium CPS using dynamic sorption technique. Fusion Engineering and Design, 2019, 146, 402-405.	1.9	13
15	Simulation of hydrogen isotopes absorption by metals under uncompensated pressure conditions. International Journal of Hydrogen Energy, 2019, 44, 29304-29309.	7.1	9
16	Study of tritium and helium generation and release from lead-lithium eutectics Li15.7Pb under neutron irradiation. Fusion Engineering and Design, 2019, 146, 1317-1320.	1.9	11
17	Reactor experiments to study luminescence of He-Ne and He-Kr gaseous mixtures, excited by the products of 6Li (n, Î±) 3H nuclear reaction. , 2018, , .		2
18	Lithium CPS based on carboxylic fabric with CNT synthesized on its fibersâ€™ surface. Materials Today: Proceedings, 2017, 4, 4524-4533.	1.8	1

#	Article	IF	Citations
19	Study of properties of tungsten irradiated in hydrogen atmosphere. Nuclear Fusion, 2017, 57, 126062.	3.5	15
20	Investigation of hydrogen isotopes interaction with lithium CPS under reactor irradiation. Fusion Engineering and Design, 2017, 124, 324-327.	1.9	5
21	â€œIsotope effectâ€•of hydrogen and deuterium interaction with vanadium alloys VCrTi. Materials Today: Proceedings, 2017, 4, 4582-4588.	1.8	1
22	Study of processes of nuclear reaction energy conversion into energy of optical radiation. Materials Today: Proceedings, 2017, 4, 4589-4598.	1.8	2
23	Development of the reactor lithium ampoule device for research of spectral-luminescent characteristics of nuclear-excited plasma. Fusion Engineering and Design, 2017, 117, 204-207.	1.9	10
24	Results of neutron irradiation of liquid lithium saturated with deuterium. Fusion Engineering and Design, 2017, 117, 194-198.	1.9	16
25	Development of technology for fabrication of lithium CPS on basis of CNT-reinforced carboxylic fabric. Fusion Engineering and Design, 2017, 117, 168-174.	1.9	3
26	Studies on gas release from pre-saturated samples on a plasma beam installation. Physical Sciences and Technology, 2017, 4, 15-28.	0.2	0
27	Experimental facility for reactor experiments on study of spectral-luminescent characteristics of nuclear-excited plasma. Journal of Physics: Conference Series, 2016, 747, 012012.	0.4	3
28	Investigation of hydrogen isotopes interaction processes with lithium under neutron irradiation. Fusion Engineering and Design, 2016, 109-111, 26-29.	1.9	6
29	Early evaluation of hydrogen isotopes separation by V4Cr4Ti-based sorbents at low temperatures. Fusion Engineering and Design, 2016, 113, 303-307.	1.9	6
30	Determination of tritium generation and release parameters at lithium CPS under neutron irradiation. Fusion Engineering and Design, 2016, 109-111, 52-56.	1.9	6
31	Investigation of parameters of interaction of hydrogen isotopes with liquid lithium and lithium capillary-porous system under reactor irradiation. Physics of Atomic Nuclei, 2015, 78, 1075-1086.	0.4	5
32	Determination of tritium diffusion coefficients in irradiated beryllium of S-200F grade. International Journal of Mathematics and Physics, 2015, 6, 92-97.	0.2	0
33	Interaction of tritium and helium with leadâ€“lithium eutectic under reactor irradiation. Fusion Engineering and Design, 2014, 89, 1486-1490.	1.9	16
34	Properties of tritium/helium release from hot isostatic pressed beryllium of various trademarks. Journal of Nuclear Materials, 2014, 452, 41-45.	2.7	6
35	DETRITIATION OF DIFFERENT IRRADIATED BERYLLIUM GRADES USING HIGH-TEMPERATURE DEGASSING METHOD. Problems of Atomic Science and Technology, Series Thermonuclear Fusion, 2014, 37, 27-37.	0.2	2
36	Tritium migration in the materials proposed for fusion reactors: Li2TiO3 and beryllium. Journal of Nuclear Materials, 2013, 442, S740-S745.	2.7	24

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37	Temperature dependence of the rate constant of hydrogen isotope interactions with a lithium capillary-porous system under reactor irradiation. Fusion Engineering and Design, 2013, 88, 1731-1734.	1.9	6
38	Status of design and experimental activity on module of lithium divertor for KTM tokamak. Fusion Engineering and Design, 2013, 88, 1862-1865.	1.9	22
39	THE INVESTIGATION OF HYDROGEN ISOTOPES INTERACTION PARAMETERS WITH LIQUID LITHIUM AND LITHIUM CAPILLARY-POROUS SYSTEM UNDER REACTOR IRRADIATION. Problems of Atomic Science and Technology, Series Thermonuclear Fusion, 2013, 36, 25-38.	0.2	0
40	Research of Reactor Radiation Influence upon Processes of Hydrogen Isotopes Interaction with Materials of the Fusion Facility. Fusion Science and Technology, 2011, 60, 9-15.	1.1	21
41	Tritium accumulation and release from Li2TiÐž3 during long-term irradiation in the WWR-K reactor. Journal of Nuclear Materials, 2011, 417, 748-752.	2.7	19
42	Study of Tritium and Helium Release from Irradiated Lithium Ceramics Li₂TiO₃. Fusion Science and Technology, 2011, 60, 1139-1142.	1.1	13
43	Material science activities for fusion reactors in Kazakhstan. Journal of Nuclear Materials, 2009, 386-388, 15-18.	2.7	11
44	Study of Li2TiO3+5mol% TiO2 lithium ceramics after long-term neutron irradiation. Journal of Nuclear Materials, 2009, 386-388, 286-289.	2.7	21
45	DIFFUSION OF TRITIUM GENERATED IN LITHIUM METATITANATE Li2TiO3 DURING THERMAL NEUTRON IRRADIATION IN REACTOR WWR-K. Problems of Atomic Science and Technology, Series Thermonuclear Fusion, 2009, 32, 83-92.	0.2	1
46	In-pile tritium permeation through F82H steel with and without a ceramic coating of Cr2O3â€“SiO2 including CrPO4. Fusion Engineering and Design, 2007, 82, 2246-2251.	1.9	26
47	Measurement system for in-pile tritium monitoring from Li2TiO3 ceramics at WWRK reactor. Journal of Nuclear Materials, 2007, 367-370, 1028-1032.	2.7	10
48	Studies of reactor irradiation effect on hydrogen isotope release from vanadium alloy V4Cr4Ti. Journal of Nuclear Materials, 2007, 367-370, 844-847.	2.7	7
49	Investigation of hydrogen isotope permeation through F82H steel with and without a ceramic coating of Cr2O3â€“SiO2 including CrPO4 (out-of-pile tests). Fusion Engineering and Design, 2006, 81, 701-705.	1.9	39
50	In-Pile Assemblies for Investigation of Tritium Release from Li2TiO3 Lithium Ceramic. Fusion Science and Technology, 2005, 47, 1084-1088.	1.1	7
51	Gas driven deuterium permeation through F82H martensitic steel. Journal of Nuclear Materials, 2002, 307-311, 1494-1497.	2.7	18
52	Surface Effects in Diffusion Measurements: Deuterium Permeation through Martensitic Steel. Physica Scripta, 2001, T94, 121.	2.5	17
53	Hydrogen permeation through vanadium alloy Vâ€“4Crâ€“4Ti â€˜in situâ€™ of reactor irradiation. Journal of Nuclear Materials, 2000, 283-287, 872-875.	2.7	6
54	Out of Pile Experiments on the Investigation of Hydrogen Interaction With Reduced Activation Ferritic-Martensitic Steel F82H.. , 2000, , 307-312.		0

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55	Investigation of the Surface Element Composition Influence on Hydrogen Permeability through Vanadium Alloy VCr4Ti4. Fusion Science and Technology, 1998, 34, 868-871.	0.6	12
56	Investigation of Hydrogen Permeability through Copper Alloy CUCR1ZR0.1 and Duplex Structure Be-Cu. Fusion Science and Technology, 1998, 34, 919-923.	0.6	2