

Miao Song

List of Publications by Year in descending order

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48
papers

1,495
citations

279701

23
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38
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all docs

52
docs citations

52
times ranked

1173
citing authors

#	ARTICLE	IF	CITATIONS
1	Microstructure and strengthening mechanisms in Cu/Fe multilayers. <i>Acta Materialia</i> , 2012, 60, 6312-6321.	3.8	104
2	On the stress corrosion crack growth behaviour in high temperature water of 316L stainless steel made by laser powder bed fusion additive manufacturing. <i>Corrosion Science</i> , 2017, 128, 140-153.	3.0	104
3	In situ Evidence of Defect Cluster Absorption by Grain Boundaries in Kr Ion Irradiated Nanocrystalline Ni. <i>Metallurgical and Materials Transactions A: Physical Metallurgy and Materials Science</i> , 2013, 44, 1966-1974.	1.1	103
4	In Situ Study of Defect Migration Kinetics and Self-Healing of Twin Boundaries in Heavy Ion Irradiated Nanotwinned Metals. <i>Nano Letters</i> , 2015, 15, 2922-2927.	4.5	90
5	Radiation damage and irradiation-assisted stress corrosion cracking of additively manufactured 316L stainless steels. <i>Journal of Nuclear Materials</i> , 2019, 513, 33-44.	1.3	89
6	Response of equal channel angular extrusion processed ultrafine-grained T91 steel subjected to high temperature heavy ion irradiation. <i>Acta Materialia</i> , 2014, 74, 285-295.	3.8	78
7	A roadmap for tailoring the strength and ductility of ferritic/martensitic T91 steel via thermo-mechanical treatment. <i>Acta Materialia</i> , 2016, 112, 361-377.	3.8	76
8	Unusual size-dependent strengthening mechanisms in helium ion-irradiated immiscible coherent Cu/Co nanolayers. <i>Acta Materialia</i> , 2015, 84, 393-404.	3.8	75
9	Insights into the stress corrosion cracking of solution annealed alloy 690 in simulated pressurized water reactor primary water under dynamic straining. <i>Acta Materialia</i> , 2018, 151, 321-333.	3.8	66
10	Tailoring the strength and ductility of T91 steel by partial tempering treatment. <i>Acta Materialia</i> , 2019, 169, 209-224.	3.8	59
11	Microstructure refinement and strengthening mechanisms of a 12Cr ODS steel processed by equal channel angular extrusion. <i>Journal of Alloys and Compounds</i> , 2013, 577, 247-256.	2.8	52
12	Enhanced void swelling in NiCoFeCrPd high-entropy alloy by indentation-induced dislocations. <i>Materials Research Letters</i> , 2018, 6, 584-591.	4.1	46
13	The oxidation of alloy 690 in simulated pressurized water reactor primary water. <i>Corrosion Science</i> , 2017, 126, 227-237.	3.0	44
14	Enhancement of strength and ductility in ultrafine-grained T91 steel through thermomechanical treatments. <i>Journal of Materials Science</i> , 2013, 48, 7360-7373.	1.7	43
15	Enhanced radiation tolerance in immiscible Cu/Fe multilayers with coherent and incoherent layer interfaces. <i>Journal of Materials Research</i> , 2015, 30, 1300-1309.	1.2	34
16	Probing long-range ordering in nickel-base alloys with proton irradiation. <i>Acta Materialia</i> , 2018, 156, 446-462.	3.8	33
17	Characterization of alloy 718 subjected to different thermomechanical treatments. <i>Materials Science & Engineering A: Structural Materials: Properties, Microstructure and Processing</i> , 2017, 691, 195-202.	2.6	30
18	On the Thermal Stability of Dislocation Cellular Structures in Additively Manufactured Austenitic Stainless Steels: Roles of Heavy Element Segregation and Stacking Fault Energy. <i>Jom</i> , 2020, 72, 4232-4243.	0.9	28

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19	Radiation tolerance of commercial and advanced alloys for core internals: a comprehensive microstructural characterization. <i>Journal of Nuclear Materials</i> , 2018, 510, 396-413.	1.3	27
20	In situ studies of radiation induced crystallization in Fe/a-Y2O3 nanolayers. <i>Journal of Nuclear Materials</i> , 2014, 452, 321-327.	1.3	26
21	Irradiation assisted stress corrosion cracking of commercial and advanced alloys for light water reactor core internals. <i>Journal of Nuclear Materials</i> , 2019, 515, 52-70.	1.3	26
22	Phase constitution effect on the ductility of low alloy multiphase transformation induced plasticity steels. <i>Materials Science & Engineering A: Structural Materials: Properties, Microstructure and Processing</i> , 2013, 569, 137-143.	2.6	25
23	Grain refinement mechanisms and strength-hardness correlation of ultra-fine grained grade 91 steel processed by equal channel angular extrusion. <i>International Journal of Pressure Vessels and Piping</i> , 2019, 172, 212-219.	1.2	25
24	<i>In situ</i> Observation of Defect Annihilation in Kr Ion-Irradiated Bulk Fe/Amorphous-Fe ₂ Zr Nanocomposite Alloy. <i>Materials Research Letters</i> , 2015, 3, 35-42.	4.1	20
25	Amyloid-like amelogenin nanoribbons template mineralization via a low-energy interface of ion binding sites. <i>Proceedings of the National Academy of Sciences of the United States of America</i> , 2022, 119, e2106965119.	3.3	19
26	Tunable mechanical property and strain hardening behavior of a single-phase CoFeNi ₂ V _{0.5} Mo _{0.2} high entropy alloy. <i>Materials Science & Engineering A: Structural Materials: Properties, Microstructure and Processing</i> , 2020, 776, 139027.	2.6	16
27	In situ studies on superior thermal stability of bulk FeZr nanocomposites. <i>Acta Materialia</i> , 2015, 101, 125-135.	3.8	14
28	Irradiation-Induced Extremes Create Hierarchical Face-Centered-Cubic Phases in Nanostructured High Entropy Alloys. <i>Advanced Materials</i> , 2020, 32, 2002652.	11.1	14
29	Effect of deformation level and orientation on SCC of 316L stainless steel in simulated light water environments. <i>Journal of Nuclear Materials</i> , 2020, 531, 152038.	1.3	14
30	A comparison study of void swelling in additively manufactured and cold-worked 316L stainless steels under ion irradiation. <i>Journal of Nuclear Materials</i> , 2021, 551, 152946.	1.3	14
31	A high-resolution characterization of irradiation-assisted stress corrosion cracking of proton-irradiated 316L stainless steel in simulated pressurized water reactor primary water. <i>Corrosion Science</i> , 2022, 199, 110187.	3.0	14
32	Atomic scale structure dominated FCC and B2 responses to He ion irradiation in eutectic high-entropy alloy AlCoCrFeNi _{2.1} . <i>Journal of Materials Science and Technology</i> , 2022, 129, 87-95.	5.6	13
33	Elastic strain energy control of the precipitate free zone around primary carbides in nickel base alloy 725. <i>Acta Materialia</i> , 2016, 120, 138-149.	3.8	12
34	Microstructural characterization of cold-worked 316 stainless steel flux thimble tubes irradiated up to 100 dpa in a commercial Pressurized Water Reactor. <i>Journal of Nuclear Materials</i> , 2020, 541, 152400.	1.3	11
35	Insight into the acceleration in oxidation kinetics ahead of stress corrosion crack of alloy 690 in simulated PWR primary water. <i>Corrosion Science</i> , 2020, 176, 108943.	3.0	9
36	Effects of heat treatment on corrosion fatigue and stress corrosion crack growth of additive-manufactured Alloy 800H in high-temperature water. <i>Corrosion Science</i> , 2021, 191, 109739.	3.0	9

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37	Compositionally graded specimen made by laser additive manufacturing as a high-throughput method to study radiation damages and irradiation-assisted stress corrosion cracking. <i>Journal of Nuclear Materials</i> , 2022, 560, 153493.	1.3	9
38	Microstructure and tensile behavior of small scale resistance spot welded sandwich bulk metallic glasses. <i>Journal of Non-Crystalline Solids</i> , 2016, 447, 300-306.	1.5	6
39	Surface hardening of metals at room temperature by nanoparticle-laden cavitating waterjets. <i>Journal of Materials Processing Technology</i> , 2020, 275, 116316.	3.1	6
40	Comparison of Modified Mohr's-Coulomb Model and Bai's-Wierzbicki Model for Constructing 3D Ductile Fracture Envelope of AA6063. <i>Chinese Journal of Mechanical Engineering (English Edition)</i> , 2021, 34, .	1.9	3
41	A microscopic and crystallographic study of proton irradiated alloy 718. <i>Journal of Nuclear Materials</i> , 2021, 551, 152954.	1.3	3
42	Sensitization, desensitization, and carbide evolution of Alloy 800H made by laser powder bed fusion. <i>Additive Manufacturing</i> , 2022, 50, 102547.	1.7	2
43	Microstructural Study on the Stress Corrosion Cracking of Alloy 690 in Simulated Pressurized Water Reactor Primary Environment. <i>Minerals, Metals and Materials Series</i> , 2018, , 535-545.	0.3	1
44	Microstructural Study on the Stress Corrosion Cracking of Alloy 690 in Simulated Pressurized Water Reactor Primary Environment. <i>Minerals, Metals and Materials Series</i> , 2019, , 535-545.	0.3	1
45	Stress Corrosion Cracking Behavior of Alloy 718 Subjected to Various Thermal Mechanical Treatments in Primary Water. <i>Minerals, Metals and Materials Series</i> , 2018, , 293-305.	0.3	0
46	Advanced Characterization of Additively Manufactured 316L Stainless Steel for Nuclear Applications. <i>Microscopy and Microanalysis</i> , 2021, 27, 2160-2161.	0.2	0
47	Irradiation Assisted Stress Corrosion Cracking (IASCC) of Nickel-Base Alloys in Light Water Reactors Environments Part II: Stress Corrosion Cracking. <i>Minerals, Metals and Materials Series</i> , 2018, , 961-972.	0.3	0
48	High-Entropy Alloys: Irradiation-Induced Extremes Create Hierarchical Face-Centered-Cubic Phases in Nanostructured High Entropy Alloys (<i>Adv. Mater.</i> 39/2020). <i>Advanced Materials</i> , 2020, 32, .	11.1	0