Hsoung-Wei Chou

List of Publications by Year
in descending order

29 papers	91 citations	1684188 5 h-index	1474206 9 g-index
29 all docs	29 docs citations	29 times ranked	41 citing authors

#	Article	IF	Citations
1	Application of Flaw Updating Process on Probabilistic Integrity Analysis for a Reactor Pressure Vessel Subjected to Pressurized Thermal Shocks. Nuclear Technology, 2021, 207, 735-749.	1.2	0
2	Probabilistic fracture analysis for boiling water reactor vessels considering seismic loads during decommissioning transition period. Annals of Nuclear Energy, 2021, , 108827.	1.8	1
3	Development of pressure-temperature operation limits for a PWR vessel considering beltline shell and extended beltline nozzles. International Journal of Pressure Vessels and Piping, 2020, 179, 103944.	2.6	5
4	Large thermal gradients on structural integrity of a reactor pressure vessel subjected to pressurized thermal shocks. International Journal of Pressure Vessels and Piping, 2020, 179, 103942.	2.6	3
5	Demonstration of Structural Integrity of Boiling Water Reactor Pressure Vessels Under Ultimate Response Guideline Operation. Nuclear Technology, 2020, 206, 1919-1931.	1.2	2
6	Flaw Acceptance Evaluation for the Final Disposal Canister Under Earthquake Induced Rock Shear. , 2019, , .		0
7	The Optimization Design of Storage Efficiency and Structural Analysis for the 3 Cubic Meter Radioactive Waste Container. , 2019, , .		0
8	Comparison of Pressure-Temperature Limits for a Pressurized Water Reactor Pressure Vessel Considering Beltline and Extended Beltline Regions. , 2018, , .		1
9	Application of Flaw Updating Process on Probabilistic Structural Evaluation for a Reactor Pressure Vessel Under Pressurized Thermal Shocks. , 2018, , .		0
10	Comparison of ASME pressureâ€"temperature limits on the fracture probability for a pressurized water reactor pressure vessel. Annals of Nuclear Energy, 2017, 108, 366-375.	1.8	7
11	Fracture Probability Assessment for Embrittled Reactor Pressure Vessels Under Ultimate Response Guideline Operation. , 2016, , .		0
12	Fracture risk assessment for the pressurized water reactor pressure vessel under pressurized thermal shock events. Nuclear Engineering and Design, 2016, 300, 412-421.	1.7	9
13	Probabilistic ageing and risk analysis tools for nuclear piping. Nuclear Engineering and Design, 2016, 300, 541-551.	1.7	4
14	Application of Probabilistic Fracture Mechanics Analysis on BWR Recirculation Piping Systems. , 2016, , .		0
15	The influence of chemistry concentration on the fracture risk of a reactor pressure vessel subjected to pressurized thermal shocks. Nuclear Engineering and Design, 2016, 297, 188-196.	1.7	2
16	Probabilistic Structural Integrity Analysis of Boiling Water Reactor Pressure Vessel under Low Temperature Overpressure Event. International Journal of Nuclear Energy, 2015, 2015, 1-9.	0.4	4
17	Probabilistic Structural Integrity Evaluation on a Pressurized Water Reactor Pressure Vessel Under Pressureâ€"Temperature Limit Operations. , 2015, , .		0
18	Effects of Chemical Content Variation on the Fracture Probability of PWR Pressure Vessel Subjected to PTS Events. , 2015, , .		0

#	Article	IF	Citations
19	Effects of fracture toughness curves of ASME Section XIâ€“Appendix G on a reactor pressure vessel under pressureâ€“temperature limit operation. Nuclear Engineering and Design, 2014, 280, 404-412.	1.7	22
20	Reactor pressure vessel integrity assessment by probabilistic fracture mechanics â€“ A plant specific analysis. International Journal of Pressure Vessels and Piping, 2014, 117-118, 64-69.	2.6	2
21	Probabilistic Fracture Mechanics Analysis for Degraded Reactor Pressure Vessel in Pressurized Water Reactor Nuclear Power Plant. , 2014, , .		1
22	Structural Reliability Evaluation on the Pressurized Water Reactor Pressure Vessel Under Pressurized Thermal Shock Events. , 2014, , .		5
23	Effects of Fracture Toughness Curves of ASME Section XI: Appendix G on a Reactor Pressure Vessel Under Pressureâ€“Temperature Limit Operation. , 2013, , .		0
24	Failure Probability Assessment for a Boiling Water Reactor Pressure Vessel Under Low Temperature Over-Pressure Event. , 2012, , .		1
25	Probabilistic fracture analysis for boiling water reactor pressure vessels subjected to low temperature over-pressure event. Annals of Nuclear Energy, 2012, 43, 61-67.	1.8	11
26	Fatigue life prediction for circular rubber bearings subjected to cyclic compression. Journal of Applied Polymer Science, 2012, 123, 2194-2203.	2.6	5
27	Crack initiation and propagation in circular rubber bearings subjected to cyclic compression. Journal of Applied Polymer Science, 2011, 121, 1747-1756.	2.6	5
28	Probabilistic Fracture Analysis for Boiling Water Reactor Pressure Vessels Subjected to Low Temperature Over-Pressure Event. , 2010, , .		0
29	Boiling Water Reactor Pressure Vessel Integrity Evaluation by Probabilistic Fracture Mechanics. , 2010, , .		1