

# Steven J Zinkle

## List of Publications by Year in descending order

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297  
papers

20,492  
citations

15001

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14012

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311  
docs citations

311  
times ranked

9221  
citing authors

#	ARTICLE	IF	CITATIONS
1	Toward accurate evaluation of bulk hardness from nanoindentation testing at low indent depths. <i>Materials and Design</i> , 2022, 213, 110317.	3.3	24
2	Radiation induced amorphization of carbides in additively manufactured and conventional ferritic-martensitic steels: In-situ experiments on extraction replicas. <i>Journal of Nuclear Materials</i> , 2022, 563, 153646.	1.3	6
3	Phase instabilities in austenitic steels during particle bombardment at high and low dose rates. <i>Materials and Design</i> , 2022, 217, 110588.	3.3	6
4	Properties and microstructure evolution of silicon nitride and zirconium nitride following Ni ion irradiation. <i>Journal of Nuclear Materials</i> , 2022, 563, 153643.	1.3	5
5	Effect of heavy ion irradiation dose rate and temperature on $\delta$ -Fe <sub>2</sub> O <sub>3</sub> precipitation in high purity Fe-18%Cr alloy. <i>Acta Materialia</i> , 2022, 231, 117888.	3.8	9
6	Irradiation damage concurrent challenges with RAFM and ODS steels for fusion reactor first-wall/blanket: a review. <i>JPhys Energy</i> , 2022, 4, 034003.	2.3	9
7	The effect of helium on cavity swelling in dual-ion irradiated Fe and Fe-10Cr ferritic alloys. <i>Journal of Nuclear Materials</i> , 2022, 569, 153907.	1.3	6
8	An exploratory study on helium mobility in amorphous and crystallized bulk metallic glasses. <i>Journal of Nuclear Materials</i> , 2021, 543, 152617.	1.3	2
9	Interdiffusion of Elements During Ultrasonic Additive Manufacturing. <i>Metallurgical and Materials Transactions A: Physical Metallurgy and Materials Science</i> , 2021, 52, 1142-1157.	1.1	11
10	Development of the materials analysis and particle probe for Proto-MPEX. <i>Review of Scientific Instruments</i> , 2021, 92, 045108.	0.6	2
11	Temperature-dependent cavity swelling in dual-ion irradiated Fe and Fe-Cr ferritic alloys. <i>Acta Materialia</i> , 2021, 207, 116660.	3.8	41
12	Effect of sink strength on coherency loss of precipitates in dilute Cu-base alloys during in situ ion irradiation. <i>Acta Materialia</i> , 2021, 210, 116812.	3.8	6
13	Characterization of high thermal conductivity fuel surrogates before and after ion irradiation. <i>Journal of Nuclear Materials</i> , 2021, 552, 153027.	1.3	5
14	Tailoring microstructure in sintered Cu-Cr-Nb-Zr alloys for fusion components. <i>Journal of Nuclear Materials</i> , 2021, 551, 152956.	1.3	8
15	On the use of SRIM for calculating vacancy production: Quick calculation and full-cascade options. <i>Nuclear Instruments &amp; Methods in Physics Research B</i> , 2021, 503, 11-29.	0.6	70
16	RF sheath induced sputtering on Proto-MPEX. I. Sheath equivalent dielectric layer for modeling the RF sheath. <i>Physics of Plasmas</i> , 2021, 28, .	0.7	8
17	Perspectives on multiscale modelling and experiments to accelerate materials development for fusion. <i>Journal of Nuclear Materials</i> , 2021, 554, 153113.	1.3	37
18	Helium causing disappearance of $\delta$ -Fe <sub>2</sub> O <sub>3</sub> dislocation loops in binary Fe-Cr ferritic alloys. <i>Journal of Nuclear Materials</i> , 2021, 556, 153213.	1.3	2

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19	Reconfiguration of an Electrothermal-Arc Plasma Source for In Situ PMI Studies. <i>Fusion Science and Technology</i> , 2021, 77, 921-927.	0.6	2
20	RF sheath induced sputtering on Proto-MPEX part 2: Impurity transport modeling and experimental comparison. <i>Physics of Plasmas</i> , 2021, 28, 103508.	0.7	6
21	Irradiation hardening and ductility loss of Eurofer97 steel variants after neutron irradiation to ITER-TBM relevant conditions. <i>Fusion Engineering and Design</i> , 2021, 173, 112935.	1.0	5
22	Investigation of the mechanical and microstructural evolution of a Cu based bulk metallic glass during ion irradiation. <i>Intermetallics</i> , 2020, 116, 106655.	1.8	13
23	Neutron irradiation-induced microstructure damage in ultra-high temperature ceramic TiC. <i>Acta Materialia</i> , 2020, 186, 1-10.	3.8	30
24	Modern nanostructured ferritic alloys: A compelling and viable choice for sodium fast reactor fuel cladding applications. <i>Journal of Nuclear Materials</i> , 2020, 529, 151928.	1.3	21
25	Effects of carbonitrides and carbides on microstructure and properties of castable nanostructured alloys. <i>Journal of Nuclear Materials</i> , 2020, 540, 152376.	1.3	21
26	Effects of irradiation spectrum on the microstructural and mechanical properties of bulk metallic glasses. <i>Journal of Nuclear Materials</i> , 2020, 533, 152084.	1.3	4
27	Observed volatilization behavior of silicon carbide in flowing hydrogen above 2000 K. <i>Journal of the American Ceramic Society</i> , 2020, 103, 7159-7171.	1.9	2
28	The effects of neutron and ionizing irradiation on the aqueous corrosion of SiC. <i>Journal of Nuclear Materials</i> , 2020, 536, 152190.	1.3	17
29	The effect of Zr on precipitation in oxide dispersion strengthened FeCrAl alloys. <i>Journal of Nuclear Materials</i> , 2020, 533, 152105.	1.3	21
30	Structural damage and phase stability of Al <sub>0.3</sub> CoCrFeNi high entropy alloy under high temperature ion irradiation. <i>Acta Materialia</i> , 2020, 188, 1-15.	3.8	83
31	Contrasting roles of Laves_Cr <sub>2</sub> Nb precipitates on the creep properties of novel CuCrNbZr alloys. <i>Materials Science &amp; Engineering A: Structural Materials: Properties, Microstructure and Processing</i> , 2020, 779, 139110.	2.6	16
32	Effects of temperature on helium bubble behaviour in Fe-9Cr alloy. <i>Journal of Nuclear Materials</i> , 2020, 532, 152045.	1.3	20
33	Experimental Studies on Primary Damage Formation. , 2020, , 74-90.		3
34	Toward the Use of Ion Irradiation to Predict Reactor Irradiation Effects. , 2020, , 468-484.		11
35	Radiation-Induced Effects on Microstructure. , 2020, , 91-129.		17
36	Radiation Effects in Copper and Copper Alloys for Fusion Applications. , 2020, , 93-113.		5

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37	Cavity Swelling in Irradiated Materials. , 2020, , 406-455.		10
38	Evolution of the microstructural and mechanical properties of BAM-11 bulk metallic glass during ion irradiation and annealing. Journal of Nuclear Materials, 2019, 523, 299-309.	1.3	29
39	Neutron irradiation response of a Co-free high entropy alloy. Journal of Nuclear Materials, 2019, 527, 151838.	1.3	64
40	Investigation of the thermal and neutron irradiation response of BAM-11 bulk metallic glass. Journal of Nuclear Materials, 2019, 526, 151771.	1.3	30
41	Radiation and Thermomechanical Degradation Effects in Reactor Structural Alloys. , 2019, , 163-210.		5
42	Stability of a model Fe-14Cr nanostructured ferritic alloy after long-term thermal creep. Scripta Materialia, 2019, 170, 134-139.	2.6	11
43	Characterization of the helicon plasma flux to the target of Proto-MPEX. Fusion Engineering and Design, 2019, 138, 282-288.	1.0	7
44	Microstructural evaluation of a Fe-12Cr nanostructured ferritic alloy designed for impurity sequestration. Journal of Nuclear Materials, 2019, 522, 111-122.	1.3	9
45	Ionizing vs collisional radiation damage in materials: Separated, competing, and synergistic effects in Ti3SiC2. Acta Materialia, 2019, 173, 195-205.	3.8	10
46	First principle study of magnetism and vacancy energetics in a near equimolar NiFeMnCr high entropy alloy. Journal of Applied Physics, 2019, 125, .	1.1	40
47	Entropy modeling on serrated flows in carburized steels. Materials Science & Engineering A: Structural Materials: Properties, Microstructure and Processing, 2019, 753, 135-145.	2.6	20
48	Multiscale investigations of nanoprecipitate nucleation, growth, and coarsening in annealed low-Cr oxide dispersion strengthened FeCrAl powder. Acta Materialia, 2019, 166, 1-17.	3.8	46
49	Towards a greater understanding of serrated flows in an Al-containing high-entropy-based alloy. International Journal of Plasticity, 2019, 115, 71-92.	4.1	89
50	Nano-scale microstructure damage by neutron irradiations in a novel Boron-11 enriched TiB2 ultra-high temperature ceramic. Acta Materialia, 2019, 165, 26-39.	3.8	28
51	Post irradiation examination of nanoprecipitate stability and $\text{L}_{12}$ precipitation in an oxide dispersion strengthened Fe-12Cr-5Al alloy. Scripta Materialia, 2019, 162, 94-98.	2.6	19
52	Roadmap for the application of ion beam technologies to the challenges of nuclear energy technologies. Nuclear Instruments & Methods in Physics Research B, 2019, 441, 41-45.	0.6	17
53	Comparison of irradiation tolerance of two MAX phases-Ti4AlN3 and Ti2AlN. Journal of Nuclear Materials, 2019, 513, 120-128.	1.3	7
54	Helium induced microstructure damage, nano-scale grain formation and helium retention behaviour of ZrC. Acta Materialia, 2019, 163, 14-27.	3.8	31

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55	Improving atomic displacement and replacement calculations with physically realistic damage models. Nature Communications, 2018, 9, 1084.	5.8	241
56	Opportunities and limitations for ion beams in radiation effects studies: Bridging critical gaps between charged particle and neutron irradiations. Scripta Materialia, 2018, 143, 154-160.	2.6	229
57	Effects of temperature on the irradiation responses of Al <sub>0.1</sub> CoCrFeNi high entropy alloy. Scripta Materialia, 2018, 144, 31-35.	2.6	103
58	Modeling of dislocation channel width evolution in irradiated metals. Journal of Nuclear Materials, 2018, 499, 47-64.	1.3	12
59	Complexity modeling and analysis of chaos and other fluctuating phenomena. Chaos, Solitons and Fractals, 2018, 116, 166-175.	2.5	18
60	A set of MATLAB routines and associated files for prediction of radiation-enhanced diffusion in ion irradiated materials. Data in Brief, 2018, 21, 83-85.	0.5	3
61	Primary radiation damage: A review of current understanding and models. Journal of Nuclear Materials, 2018, 512, 450-479.	1.3	358
62	Influence of mechanical alloying and extrusion conditions on the microstructure and tensile properties of Low-Cr ODS FeCrAl alloys. Journal of Nuclear Materials, 2018, 512, 227-238.	1.3	37
63	Irradiation responses and defect behavior of single-phase concentrated solid solution alloys. Journal of Materials Research, 2018, 33, 3077-3091.	1.2	47
64	Thermal diffusion of mixed valence Ce in 6Li loaded silicate glass for neutron imaging. Journal of Non-Crystalline Solids, 2018, 498, 145-152.	1.5	3
65	Modeling the impact of radiation-enhanced diffusion on implanted ion profiles. Journal of Nuclear Materials, 2018, 509, 168-180.	1.3	36
66	Production and Hot Hydrogen Testing of Subscale Molybdenum Cermets for Nuclear Thermal Propulsion. , 2018, , .		2
67	Development of novel Cu-Cr-Nb-Zr alloys with the aid of computational thermodynamics. Materials and Design, 2018, 156, 370-380.	3.3	40
68	High pressure synthesis of a hexagonal close-packed phase of the high-entropy alloy CrMnFeCoNi. Nature Communications, 2017, 8, 15634.	5.8	241
69	Development of next generation tempered and ODS reduced activation ferritic/martensitic steels for fusion energy applications. Nuclear Fusion, 2017, 57, 092005.	1.6	177
70	Towards a programme of testing and qualification for structural and plasma-facing materials in "fusion neutron" environments. Nuclear Fusion, 2017, 57, 092013.	1.6	36
71	Introduction to the special issue on the technical status of materials for a fusion reactor. Nuclear Fusion, 2017, 57, 092001.	1.6	36
72	Materials testing facilities and programmes for fission and ion implantation damage. Nuclear Fusion, 2017, 57, 092011.	1.6	15

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73	Development of benchmark reduced activation ferritic/martensitic steels for fusion energy applications. Nuclear Fusion, 2017, 57, 092004.	1.6	91
74	Helium sequestration at nanoparticle-matrix interfaces in helium- $\alpha$ -heavy ion irradiated nanostructured ferritic alloys. Journal of Nuclear Materials, 2017, 483, 21-34.	1.3	42
75	Phase stability, swelling, microstructure and strength of Ti <sub>3</sub> SiC <sub>2</sub> -TiC ceramics after low dose neutron irradiation. Journal of Nuclear Materials, 2017, 483, 44-53.	1.3	31
76	Advanced irradiation-resistant materials for Generation IV nuclear reactors. , 2017, , 569-594.		11
77	Applicability of copper alloys for DEMO high heat flux components. Physica Scripta, 2016, T167, 014004.	1.2	37
78	Microstructural stability and mechanical behavior of FeNiMnCr high entropy alloy under ion irradiation. Acta Materialia, 2016, 113, 230-244.	3.8	450
79	Motivation for utilizing new high-performance advanced materials in nuclear energy systems. Current Opinion in Solid State and Materials Science, 2016, 20, 401-410.	5.6	64
80	Anisotropic swelling and microcracking of neutron irradiated Ti <sub>3</sub> AlC <sub>2</sub> -Ti <sub>5</sub> Al <sub>2</sub> C <sub>3</sub> materials. Scripta Materialia, 2016, 114, 74-78.	2.6	43
81	High temperature ion irradiation effects in MAX phase ceramics. Acta Materialia, 2016, 105, 130-146.	3.8	102
82	Superior radiation-resistant nanoengineered austenitic 304L stainless steel for applications in extreme radiation environments. Scientific Reports, 2015, 5, 7801.	1.6	82
83	Fusion materials science and technology research opportunities now and during the ITER era. Fusion Engineering and Design, 2014, 89, 1579-1585.	1.0	40
84	Accident tolerant fuels for LWRs: A perspective. Journal of Nuclear Materials, 2014, 448, 374-379.	1.3	754
85	Advanced oxidation-resistant iron-based alloys for LWR fuel cladding. Journal of Nuclear Materials, 2014, 448, 420-435.	1.3	403
86	Materials R&D for a timely DEMO: Key findings and recommendations of the EU Roadmap Materials Assessment Group. Fusion Engineering and Design, 2014, 89, 1586-1594.	1.0	120
87	Developing structural, high-heat flux and plasma facing materials for a near-term DEMO fusion power plant: The EU assessment. Journal of Nuclear Materials, 2014, 455, 277-291.	1.3	210
88	Designing Radiation Resistance in Materials for Fusion Energy. Annual Review of Materials Research, 2014, 44, 241-267.	4.3	522
89	Evaluation of high strength, high conductivity CuNiBe alloys for fusion energy applications. Journal of Nuclear Materials, 2014, 449, 277-289.	1.3	26
90	Effects of ion irradiation on Zr <sub>52.5</sub> Cu <sub>17.9</sub> Ni <sub>14.6</sub> Al <sub>10</sub> Ti <sub>5</sub> (BAM-11) bulk metallic glass. Intermetallics, 2014, 53, 62-66.	1.8	35

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91	Multimodal options for materials research to advance the basis for fusion energy in the ITER era. Nuclear Fusion, 2013, 53, 104024.	1.6	75
92	Evaluation of irradiation facility options for fusion materials research and development. Fusion Engineering and Design, 2013, 88, 472-482.	1.0	91
93	Materials challenges in nuclear energy. Acta Materialia, 2013, 61, 735-758.	3.8	1,711
94	Challenges in Developing Materials for Fusion Technology - Past, Present and Future. Fusion Science and Technology, 2013, 64, 65-75.	0.6	36
95	Opportunities and Challenges for Materials Innovation in Nuclear Energy. EPJ Web of Conferences, 2013, 51, 01001.	0.1	2
96	Radiation-Induced Effects on Microstructure**Prepared for the Oak Ridge National Laboratory under Contract No. DE-AC05-00OR22725. , 2012, , 65-98.		110
97	Effect of H and He irradiation on cavity formation and blistering in ceramics. Nuclear Instruments & Methods in Physics Research B, 2012, 286, 4-19.	0.6	99
98	Physical and Mechanical Properties of Copper and Copper Alloys. , 2012, , 667-690.		126
99	Specification of CuCrZr alloy properties after various thermo-mechanical treatments and design allowables including neutron irradiation effects. Journal of Nuclear Materials, 2011, 417, 904-907.	1.3	79
100	Prospects for accelerated development of high performance structural materials. Journal of Nuclear Materials, 2011, 417, 2-8.	1.3	89
101	Influence of thermal and radiation effects on microstructural and mechanical properties of Nbâ€“1Zr. Journal of Nuclear Materials, 2011, 414, 286-302.	1.3	12
102	Creep and fatigue issues for structural materials in demonstration fusion energy systems. Transactions of the Indian Institute of Metals, 2010, 63, 331-337.	0.7	4
103	Anomaly in dependence of radiation-induced vacancy accumulation on grain size. Journal of Nuclear Materials, 2010, 405, 261-265.	1.3	15
104	The Science and Technologies for Fusion Energy With Lasers and Direct-Drive Targets. IEEE Transactions on Plasma Science, 2010, 38, 690-703.	0.6	51
105	What is the Limit of Nanoparticle Strengthening?. MRS Bulletin, 2009, 34, 173-177.	1.7	11
106	Nb-Base FS-85 Alloy as a Candidate Structural Material for Space Reactor Applications: Effects of Thermal Aging. Metallurgical and Materials Transactions A: Physical Metallurgy and Materials Science, 2009, 40, 838-855.	1.1	16
107	Structural materials for fission & fusion energy. Materials Today, 2009, 12, 12-19.	8.3	885
108	Environmental effects on irradiation creep behavior of highly purified Vâ€“4Crâ€“4Ti alloys (NIFS-Heats) irradiated by neutrons. Journal of Nuclear Materials, 2009, 386-388, 575-578.	1.3	11

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109	Irradiation creep of the US Heat 832665 of $\text{V}\text{-}4\text{Cr}\text{-}4\text{Ti}$ . Journal of Nuclear Materials, 2009, 386-388, 618-621.	1.3	15
110	Advances in microstructural characterization. Journal of Nuclear Materials, 2009, 386-388, 8-14.	1.3	19
111	Tensile and fracture toughness properties of neutron-irradiated $\text{CuCrZr}$ . Journal of Nuclear Materials, 2009, 393, 36-46.	1.3	37
112	Technical issues of reduced activation ferritic/martensitic steels for fabrication of ITER test blanket modules. Fusion Engineering and Design, 2008, 83, 1471-1476.	1.0	84
113	Development of advanced blanket performance under irradiation and system integration through JUPITER-II project. Fusion Engineering and Design, 2008, 83, 842-849.	1.0	15
114	Low temperature neutron irradiation effects on microstructure and tensile properties of molybdenum. Journal of Nuclear Materials, 2008, 376, 11-28.	1.3	41
115	Low-temperature thermally-activated deformation and irradiation softening in neutron-irradiated molybdenum. Journal of Nuclear Materials, 2008, 377, 409-414.	1.3	10
116	Characterization of defect accumulation in neutron-irradiated Mo by positron annihilation spectroscopy. Nuclear Instruments & Methods in Physics Research B, 2008, 266, 3602-3606.	0.6	15
117	Mechanisms of stacking fault tetrahedra destruction by gliding dislocations in quenched gold. Philosophical Magazine, 2008, 88, 581-597.	0.7	54
118	Microstructures and Mechanical Properties of Irradiated Metals and Alloys. NATO Science for Peace and Security Series B: Physics and Biophysics, 2008, , 227-244.	0.2	12
119	One-Dimensional Fast Migration of Vacancy Clusters in Metals. Science, 2007, 318, 959-962.	6.0	168
120	Status of R&D activities on materials for fusion power reactors. Nuclear Fusion, 2007, 47, S696-S717.	1.6	128
121	Fracture mechanism maps in unirradiated and irradiated metals and alloys. Journal of Nuclear Materials, 2007, 361, 192-205.	1.3	18
122	Kinetics of coarsening of helium bubbles during implantation and post-implantation annealing. Journal of Nuclear Materials, 2007, 361, 149-159.	1.3	64
123	Critical questions in materials science and engineering for successful development of fusion power. Journal of Nuclear Materials, 2007, 367-370, 1-10.	1.3	67
124	Aging effects on microstructural and mechanical properties of select refractory metal alloys for space-reactor applications. Journal of Nuclear Materials, 2007, 366, 336-352.	1.3	18
125	Microstructural and mechanical property changes with aging of $\text{Mo}\text{-}41\text{Re}$ and $\text{Mo}\text{-}47.5\text{Re}$ alloys. Journal of Nuclear Materials, 2007, 366, 369-387.	1.3	18
126	Radiation-damage in molybdenum-rhenium alloys for space reactor applications. Journal of Nuclear Materials, 2007, 366, 388-406.	1.3	38



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127	Defect cluster formation and radiation hardening in molybdenum neutron-irradiated at 800°C. Journal of Nuclear Materials, 2007, 367-370, 817-822.	1.3	14
128	Biaxial thermal creep of two heats of V4Cr4Ti at 700 and 800°C in a liquid lithium environment. Journal of Nuclear Materials, 2007, 367-370, 788-793.	1.3	22
129	The temperature dependence of the yield stress for neutron-irradiated molybdenum. Journal of Nuclear Materials, 2007, 371, 53-60.	1.3	8
130	Microstructural and mechanical property changes in the Ta-base T-111 alloy following thermal aging. Journal of Nuclear Materials, 2007, 366, 353-368.	1.3	7
131	An overview of dual coolant Pb-17Li breeder first wall and blanket concept development for the US ITER-TBM design. Fusion Engineering and Design, 2006, 81, 461-467.	1.0	104
132	An overview of US ITER test blanket module program. Fusion Engineering and Design, 2006, 81, 433-441.	1.0	23
133	Destruction processes of large stacking fault tetrahedra induced by direct interaction with gliding dislocations. Journal of Nuclear Materials, 2006, 351, 285-294.	1.3	48
134	Microstructure of neutron-irradiated iron before and after tensile deformation. Journal of Nuclear Materials, 2006, 351, 269-284.	1.3	168
135	Tensile property of low activation vanadium alloy after liquid lithium exposure. Fusion Engineering and Design, 2006, 81, 307-313.	1.0	21
136	Effect of texture on the high temperature mechanical properties of Nb-1%Zr alloy. Scripta Materialia, 2006, 55, 719-722.	2.6	12
137	On the features of dislocation-obstacle interaction in thin films: large-scale atomistic simulation. Philosophical Magazine Letters, 2006, 86, 511-519.	0.5	35
138	Deformation Mechanism Maps of Unirradiated and Irradiated V-4Cr-4Ti. , 2006, , 328-344.		1
139	Overview of the U.S. Fusion Materials Sciences Program. Fusion Science and Technology, 2005, 47, 821-828.	0.6	10
140	Assessment of First Wall and Blanket Options with the Use of Liquid Breeder. Fusion Science and Technology, 2005, 47, 502-509.	0.6	27
141	Deformation microstructure of neutron-irradiated pure polycrystalline vanadium. Journal of Nuclear Materials, 2005, 336, 225-232.	1.3	33
142	Thermal conductivity degradation of ceramic materials due to low temperature, low dose neutron irradiation. Journal of Nuclear Materials, 2005, 340, 187-202.	1.3	125
143	Advanced materials for fusion technology. Fusion Engineering and Design, 2005, 74, 31-40.	1.0	173
144	Surface defects in Al2O3 and MgO irradiated with high-energy heavy ions. Surface and Coatings Technology, 2005, 196, 56-62.	2.2	25

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145	The collapse of stacking-fault tetrahedra by interaction with gliding dislocations. <i>Materials Science &amp; Engineering A: Structural Materials: Properties, Microstructure and Processing</i> , 2005, 400-401, 366-369.	2.6	35
146	Scientific Challenges, Opportunities and Priorities for the U.S. Fusion Energy Sciences Program. <i>Journal of Fusion Energy</i> , 2005, 24, 13-114.	0.5	3
147	Use of Beryllium and Beryllium Oxide in Space Reactors. <i>AIP Conference Proceedings</i> , 2005, , .	0.3	18
148	Fusion materials science: Overview of challenges and recent progress. <i>Physics of Plasmas</i> , 2005, 12, 058101.	0.7	241
149	Deformation Mechanism Maps of Unirradiated and Irradiated V-4Cr-4Ti. <i>Journal of ASTM International</i> , 2005, 2, 12462.	0.2	5
150	Radiation Effects in Refractory Alloys. <i>AIP Conference Proceedings</i> , 2004, , .	0.3	18
151	Materials needs for fusion, Generation IV fission reactors and spallation neutron sources – similarities and differences. <i>Journal of Nuclear Materials</i> , 2004, 329-333, 166-172.	1.3	262
152	Effect of initial oxygen content on the void swelling behavior of fast neutron irradiated copper. <i>Journal of Nuclear Materials</i> , 2004, 329-333, 938-941.	1.3	2
153	Deformation microstructure of neutron-irradiated pure polycrystalline metals. <i>Journal of Nuclear Materials</i> , 2004, 329-333, 947-952.	1.3	63
154	Dynamic observation of the collapse process of a stacking fault tetrahedron by moving dislocations. <i>Journal of Nuclear Materials</i> , 2004, 329-333, 919-923.	1.3	70
155	Varying temperature effects on mechanical properties of vanadium alloys during neutron irradiation. <i>Journal of Nuclear Materials</i> , 2004, 329-333, 472-476.	1.3	9
156	Materials to deliver the promise of fusion power – progress and challenges. <i>Journal of Nuclear Materials</i> , 2004, 329-333, 12-19.	1.3	129
157	Observation and analysis of defect cluster production and interactions with dislocations. <i>Journal of Nuclear Materials</i> , 2004, 329-333, 88-96.	1.3	209
158	Swift heavy ion-induced modification of Al <sub>2</sub> O <sub>3</sub> and MgO surfaces. <i>Nuclear Instruments &amp; Methods in Physics Research B</i> , 2003, 203, 136-140.	0.6	45
159	Microstructures Of Irradiated And Mechanically Deformed Metals And Alloys: Fundamental Aspects. <i>Materials Research Society Symposia Proceedings</i> , 2003, 792, 162.	0.1	4
160	Summary of the Varying Temperature Irradiation Experiment in HFIR. <i>Fusion Science and Technology</i> , 2003, 44, 450-454.	0.6	1
161	Gas Nuclear Transmutation Effects in Aluminum Nitride. <i>Fusion Science and Technology</i> , 2003, 44, 495-499.	0.6	3
162	On grain-size-dependent void swelling in pure copper irradiated with fission neutrons. <i>Philosophical Magazine A: Physics of Condensed Matter, Structure, Defects and Mechanical Properties</i> , 2002, 82, 1137-1158.	0.8	47

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163	Structural relaxation in amorphous silicon carbide. Nuclear Instruments & Methods in Physics Research B, 2002, 191, 497-503.	0.6	94
164	On the conflicting roles of ionizing radiation in ceramics. Nuclear Instruments & Methods in Physics Research B, 2002, 191, 758-766.	0.6	161
165	Impact of tritium removal and He-3 recycling on structure damage parameters in a D-T fusion system. Fusion Engineering and Design, 2002, 61-62, 561-567.	1.0	9
166	Overview of materials research for fusion reactors. Fusion Engineering and Design, 2002, 61-62, 13-25.	1.0	207
167	Nuclear features of the fusion ignition research experiment (FIRE). Fusion Engineering and Design, 2002, 63-64, 547-557.	1.0	10
168	Scientific and engineering advances from fusion materials R&D. Journal of Nuclear Materials, 2002, 307-311, 31-42.	1.3	59
169	Electrical in situ and post-irradiation properties of ceramics relevant to fusion irradiation conditions. Journal of Nuclear Materials, 2002, 307-311, 1073-1079.	1.3	7
170	Effect of periodic temperature variations on the microstructure of neutron-irradiated metals. Journal of Nuclear Materials, 2002, 307-311, 192-196.	1.3	20
171	Dose dependence of defect accumulation in neutron irradiated copper and iron. Journal of Nuclear Materials, 2002, 307-311, 912-917.	1.3	95
172	Specification of properties and design allowables for copper alloys used in HNF components of ITER. Journal of Nuclear Materials, 2002, 307-311, 668-672.	1.3	25
173	Vanadium alloys – overview and recent results. Journal of Nuclear Materials, 2002, 307-311, 547-554.	1.3	221
174	Effect of helium on the swelling of GlidCop Al25 IG alloy. Journal of Nuclear Materials, 2002, 306, 218-231.	1.3	9
175	A Plan for the Development of Fusion Energy. Journal of Fusion Energy, 2002, 21, 61-111.	0.5	20
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