

Tianyi Chen

List of Publications by Citations

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The third column is the impact factor (IF) of the journal, and the fourth column is the number of citations of the article.

25
papers

624
citations

13
h-index

24
g-index

33
ext. papers

788
ext. citations

6.1
avg. IF

3.9
L-index

#	Paper	IF	Citations
25	Defect annihilation at grain boundaries in alpha-Fe. <i>Scientific Reports</i> , 2013 , 3, 1450	4.9	69
24	Microstructural changes and void swelling of a 12Cr ODS ferritic-martensitic alloy after high-dpa self-ion irradiation. <i>Journal of Nuclear Materials</i> , 2015 , 467, 42-49	3.3	65
23	The influence of ion beam rastering on the swelling of self-ion irradiated pure iron at 450 °C. <i>Journal of Nuclear Materials</i> , 2015 , 465, 343-348	3.3	61
22	Superior radiation tolerant materials: Amorphous silicon oxycarbide. <i>Journal of Nuclear Materials</i> , 2015 , 461, 200-205	3.3	54
21	The effect of grain orientation on nanoindentation behavior of model austenitic alloy Fe-20Cr-25Ni. <i>Acta Materialia</i> , 2017 , 138, 83-91	8.4	53
20	Temperature dependent dispersoid stability in ion-irradiated ferritic-martensitic dual-phase oxide-dispersion-strengthened alloy: Coherent interfaces vs. incoherent interfaces. <i>Acta Materialia</i> , 2016 , 116, 29-42	8.4	48
19	Radiation response of alloy T91 at damage levels up to 1000 peak dpa. <i>Journal of Nuclear Materials</i> , 2016 , 482, 257-265	3.3	45
18	Stability of nanosized oxides in ferrite under extremely high dose self ion irradiations. <i>Journal of Nuclear Materials</i> , 2017 , 486, 86-95	3.3	44
17	Bifunctional nanoprecipitates strengthen and ductilize a medium-entropy alloy. <i>Nature</i> , 2021 , 595, 245-249	34.4	32
16	Effect of self-ion irradiation on the microstructural changes of alloy EK-181 in annealed and severely deformed conditions. <i>Journal of Nuclear Materials</i> , 2017 , 487, 96-104	3.3	24
15	Molecular dynamics simulations of grain boundary thermal resistance in UO ₂ . <i>Journal of Nuclear Materials</i> , 2014 , 452, 364-369	3.3	23
14	Radiation instability of equal channel angular extruded T91 at ultra-high damage levels. <i>Acta Materialia</i> , 2017 , 132, 395-404	8.4	21
13	Sink property of metallic glass free surfaces. <i>Scientific Reports</i> , 2015 , 5, 8877	4.9	17
12	High-temperature strengthening mechanisms of Laves and B ₂ precipitates in a novel ferritic alloy. <i>Materials Science & Engineering A: Structural Materials: Properties, Microstructure and Processing</i> , 2018 , 720, 110-116	5.3	11
11	Radiation response of oxide-dispersion-strengthened alloy MA956 after self-ion irradiation. <i>Nuclear Instruments & Methods in Physics Research B</i> , 2017 , 409, 259-263	1.2	9
10	Dispersoid stability in ion irradiated oxide-dispersion-strengthened alloy. <i>Journal of Nuclear Materials</i> , 2018 , 509, 504-512	3.3	8
9	Intermetallic formation and interdiffusion in diffusion couples made of uranium and single crystal iron. <i>Journal of Nuclear Materials</i> , 2015 , 467, 82-88	3.3	7

8	Phase Stability in the Fe-Rich Fe-Cr-Ni-Zr Alloys. <i>Metallurgical and Materials Transactions A: Physical Metallurgy and Materials Science</i> , 2017 , 48, 5009-5016	2.3	7
7	Temperature effect of elastic anisotropy and internal strain development in advanced nanostructured alloys: An in-situ synchrotron X-ray investigation. <i>Materials Science & Engineering A: Structural Materials: Properties, Microstructure and Processing</i> , 2017 , 692, 53-61	5.3	5
6	Interactions between displacement cascade and dislocation and their influences on Peierls stress in Fe-20Cr-25Ni alloys. <i>Computational Materials Science</i> , 2019 , 160, 279-286	3.2	5
5	The correlation between microstructure and nanoindentation property of neutron-irradiated austenitic alloy D9. <i>Acta Materialia</i> , 2020 , 195, 433-445	8.4	5
4	Coupling computational thermodynamics with density-function-theory based calculations to design L12 precipitates in Fe Ni based alloys. <i>Materials and Design</i> , 2020 , 191, 108592	8.1	4
3	Enhanced diffusion of Cr in 20Cr-25Ni type alloys under proton irradiation at 670°C. <i>Nuclear Materials and Energy</i> , 2018 , 17, 142-146	2.1	3
2	Displacement of carbon atoms in few-layer graphene. <i>Journal of Applied Physics</i> , 2020 , 128, 085902	2.5	2
1	Microstructure study of U85 wt.% Zr alloy after quick annealing at 650 °C. <i>Journal of Materials Research</i> , 2020 , 35, 1095-1102	2.5	1