Muhammad Zubair

List of Publications by Year
in descending order

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42	183	1163117	1199594
papers	citations	8 h-index	12 g-index

42	42	42	105
all docs	docs citations	times ranked	citing authors

#	Article	IF	Citations
1	Comparison of different glass materials to protect the operators from gamma-rays in the PET using MCNP code. Radiation Physics and Chemistry, 2022, 190, 109818.	2.8	28
2	Investigation of Loss of Feedwater (LOFW) Accident in the APR-1400 Using Fault Tree Analysis. Science and Technology of Nuclear Installations, 2022, 2022, 1-15.	0.8	0
3	Estimation of public exposure during normal operation of unit-1 Barakah Nuclear Power Plant using GALE and HOTSPOT. South African Journal of Chemical Engineering, 2022, 41, 235-243.	2.4	3
4	Direct Observation of Austenite and Pearlite Formation in Thermally Simulated Coarse Grain Heat-Affected Zone of Pearlite Railway Steel. Journal of Materials Engineering and Performance, 2021, 30, 497-509.	2.5	2
5	Behavior of Emergency Core Cooling System (ECCS) during the early stage of Loss of Coolant Accident (LOCA) for APR 1400 with Flownex software. Progress in Nuclear Energy, 2021, 141, 103949.	2.9	3
6	Radioactivity investigation of water and aerosols in Sharjah, United Arab Emirates. Energy Sources, Part A: Recovery, Utilization and Environmental Effects, 2019, 41, 1216-1229.	2.3	2
7	RADIATION DAMAGE EFFECTS IN OXIDE DISPERSION STRENGTHENED STEEL ALLOYS. The Proceedings of the International Conference on Nuclear Engineering (ICONE), 2019, 2019.27, 2086.	0.0	3
8	Classical molecular dynamics study for defect sink behavior in oxide dispersed strengthened alloys. , 2018, , .		2
9	Sensitivity analysis of APR-1400â€™s Reactor Protection System by using RiskSpectrum PSA. Nuclear Engineering and Design, 2018, 339, 225-234.	1.7	9
10	Analytical analysis of latent heat thermal energy storage model for solar thermal power plants. , 2017, , .		0
11	Uncertainty reduction using multivariate reliability models. , 2017, , .		0
12	Station black out concurrent with PORV failure using a Generic Pressurized Water Reactor simulator. Annals of Nuclear Energy, 2017, 110, 1081-1090.	1.8	14
13	A novel method for analytical solution of transient heat conduction and Stefan problem in cylindrical coordinate. , 2016, , .		1
14	Calculation and updating of Common Cause Failure unavailability by using alpha factor model. Annals of Nuclear Energy, 2016, 90, 106-114.	1.8	22
15	Study on nuclear accident precursors using AHP and BBN, a case study of Fukushima accident. International Journal of Energy Research, 2015, 39, 98-110.	4.5	5
16	Estimation of Surface Heat Flux and Surface Temperature during Inverse Heat Conduction under Varying Spray Parameters and Sample Initial Temperature. Scientific World Journal, The, 2014, 2014, 1-13.	2.1	3
17	A Hybrid Approach for Reliability Analysis Based on Analytic Hierarchy Process and Bayesian Network. Frontiers in Energy Research, 2014, 2, .	2.3	2
18	Reliability analysis of nuclear I&C architecture using Bayesian networks. , 2014, , .		2

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19	Study on Nuclear Accident Precursors Using AHP and BBN. Science and Technology of Nuclear Installations, 2014, 2014, 1-12.	0.8	3
20	Modeling of common cause failures (CCFs) by using beta factor parametric model. , 2014, , .		5
21	Quantitative and qualitative analysis of safety parameters in nuclear power plants. International Journal of Energy Research, 2014, 38, 755-764.	4.5	12
22	Prioritization of Underlying Precursors in Nuclear Accidents. , 2014, , .		0
23	A computer based living probabilistic safety assessment (LPSA) method for nuclear power plants. Nuclear Engineering and Design, 2013, 265, 765-771.	1.7	5
24	Reliability Data Update Method (RDUM) based on living PSA for emergency diesel generator of Daya Bay nuclear power plant. Safety Science, 2013, 59, 72-77.	4.9	13
25	Sensitivity Study on Availability of I&C Components Using Bayesian Network. Science and Technology of Nuclear Installations, 2013, 2013, 1-10.	0.8	10
26	Advancement in living probabilistic safety assessment to increase safety of nuclear power plants. Proceedings of the Institution of Mechanical Engineers, Part O: Journal of Risk and Reliability, 2013, 227, 534-539.	0.7	3
27	Role of Risk Manager for Online Risk Monitoring. Advanced Materials Research, 2011, 230-232, 424-428.	0.3	0
28	Reliability data update method for emergency diesel generator of Daya Bay Nuclear Power Plant. Annals of Nuclear Energy, 2011, 38, 2575-2580.	1.8	10
29	Calculation and Updating of Reliability Parameters in Probabilistic Safety Assessment. Journal of Fusion Energy, 2011, 30, 13-15.	1.2	13
30	A methodology for Living Probabilistic Safety Assessment (LPSA) based on Advanced Control Room Operator Support System (ACROSS). Annals of Nuclear Energy, 2011, 38, 1351-1355.	1.8	3
31	Neutronics and Thermal Hydraulic Coupling Methods for the Nuclear Reactor Core. , 2011, , .		0
32	Reliability Data Update Method with Gamma Distribution. , 2011, , .		0
33	Preliminary Assessment of Mean Steam Line Break Accident in an Integral Pressurized Water Reactor(IPWR). , 2011, , .		0
34	Estimation of Failure Rate with the Help of Risk Monitors. Lecture Notes in Electrical Engineering, 2011, , 1007-1014.	0.4	0
35	Study on the Evaluation and Simulation of Steady State Behavior and Reactor Safety Concept for Integral Pressurized Water Reactor. Information Technology Journal, 2011, 10, 983-991.	0.3	2
36	A Methodology for Updating Living Probabilistic Safety Assessment. , 2010, , .		1

#	Article	IF	Citations
37	Normal Operational State Results of Integral Pressurized Water Reactor by Using Relap5/Mod3.4. Advanced Materials Research, 2010, 171-172, 374-378.	0.3	0
38	A Review on Specific Features of Small and Medium Sized Nuclear Power Plants. , 2010, , .		2
39	Loss of Feed Water Accident in an Integral Pressurized Water Reactor (IPWR). Advanced Materials Research, 0, 171-172, 379-384.	0.3	0
40	Simulation of Loss of Coolant Accident in an Integral Pressurized Water Reactor (IPWR). Advanced Materials Research, 0, 230-232, 410-414.	0.3	0
41	Introductory Chapter: An Overview of Reliability and Risk Analysis. , 0, , .		0
42	Importance Analysis of Containment Spray System in Pressurized Water Reactor (PWR). , 0, , .		0