

# Xiaoou Yi

## List of Publications by Year in descending order

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docs citations

24  
times ranked

655  
citing authors

#	ARTICLE	IF	CITATIONS
1	High temperature annealing of ion irradiated tungsten. Acta Materialia, 2015, 90, 380-393.	7.9	162
2	Characterisation of radiation damage in W and W-based alloys from 2 MeV self-ion near-bulk implantations. Acta Materialia, 2015, 92, 163-177.	7.9	159
3	In-situ TEM studies of 150 keV W <sup>+</sup> ion irradiated W and W-alloys: Damage production and microstructural evolution. Acta Materialia, 2016, 112, 105-120.	7.9	141
4	Tungsten foil laminate for structural divertor applications – Analyses and characterisation of tungsten foil. Journal of Nuclear Materials, 2012, 424, 197-203.	2.7	63
5	Tungsten foil laminate for structural divertor applications – Tensile test properties of tungsten foil. Journal of Nuclear Materials, 2013, 434, 357-366.	2.7	51
6	Deformation behavior of austenitic stainless steel at deep cryogenic temperatures. Journal of Nuclear Materials, 2018, 504, 29-32.	2.7	39
7	Tungsten foil laminate for structural divertor applications – Joining of tungsten foils. Journal of Nuclear Materials, 2013, 436, 47-55.	2.7	30
8	Effect of trace amounts of added Sc on microstructure and mechanical properties of 2055 aluminum alloy. Materials Characterization, 2018, 141, 248-259.	4.4	30
9	Characterization of irradiation defect structures and densities by transmission electron microscopy. Journal of Materials Research, 2015, 30, 1195-1201.	2.6	29
10	A study of helium bubble production in 10 keV He <sup>+</sup> irradiated tungsten. Fusion Engineering and Design, 2017, 125, 454-457.	1.9	20
11	Microstructure evolution of beryllium with argon ion irradiation. Nuclear Materials and Energy, 2017, 13, 99-103.	1.3	12
12	High-temperature damage evolution in 10 keV He <sup>+</sup> irradiated W and W-5Re. Materials Characterization, 2018, 145, 77-86.	4.4	11
13	Impact of friction stir welding on recrystallization of oxide dispersion strengthened ferritic steel. Journal of Materials Science and Technology, 2018, 34, 209-213.	10.7	10
14	3D reconstruction of the spatial distribution of dislocation loops using an automated stereo-imaging approach. Ultramicroscopy, 2018, 195, 58-68.	1.9	10
15	Application of small specimen test technique to evaluate fracture toughness of reduced activation ferritic/martensitic steel. Fusion Engineering and Design, 2017, 125, 326-329.	1.9	9
16	High-temperature defect recovery in self-ion irradiated W-5 wt% Ta. Nuclear Materials and Energy, 2019, 18, 93-98.	1.3	9
17	Defect characterization, mechanical and thermal property evaluation in CVD-W after low-dose neutron irradiation. International Journal of Refractory Metals and Hard Materials, 2019, 85, 105004.	3.8	8
18	In-situ electron microscope observations and analysis of radiation damage in tungsten. Microscopy and Microanalysis, 2015, 21, 117-118.	0.4	3

#	ARTICLE	IF	CITATIONS
19	Interface Characteristics of Ti-Clad Vâ€“4Crâ€“4Ti Alloy Diffusion-Bonded Joint Produced by Hot Forging. Applied Sciences (Switzerland), 2018, 8, 577.	2.5	3
20	Heavy-ion irradiation and post-irradiation annealing effects in explosion-welded CuCrZr/316LN joints for ITER application. Materials Characterization, 2021, 178, 111252.	4.4	3
21	An approximate in-situ method for investigating irradiation damage of grain boundary. Nuclear Materials and Energy, 2021, 29, 101056.	1.3	2
22	Effect of Mn addition on the formation of vacancy-type dislocation loops in Î±-Fe. Materials Characterization, 2022, 185, 111755.	4.4	2
23	In-situ annealing of self-ion irradiation damage in tungsten. Materials Research Society Symposia Proceedings, 2014, 1712, 33.	0.1	1
24	Neutron irradiation response of explosion-welded CuCrZr/316LN joints for ITER application. Fusion Engineering and Design, 2021, 169, 112620.	1.9	1