

Toshihiro Yamamoto

List of Publications by Year in descending order

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#	ARTICLE	IF	CITATIONS
1	Deterministic and stochastic methods for sensitivity analysis of neutron noise. Progress in Nuclear Energy, 2022, 145, 104130.	2.9	0
2	Higher harmonic analyses of the Rossi- $\hat{\lambda}$ method and application of dynamic mode decomposition for time decay constant determination in a 1D subcritical system. Annals of Nuclear Energy, 2022, 168, 108886.	1.8	4
3	Monte Carlo sensitivity calculation in fixed source problems with the derivative source method. Journal of Computational Physics, 2022, 460, 111155.	3.8	2
4	Dynamic mode decomposition application to dominance ratio assessment in Monte Carlo k-eigenvalue calculation. Annals of Nuclear Energy, 2022, 175, 109205.	1.8	1
5	Frequency domain Monte Carlo simulations of void velocity measurements in an actual experimental setup using a neutron noise technique. Journal of Nuclear Science and Technology, 2021, 58, 190-200.	1.3	4
6	Monte Carlo perturbation calculation for geometry change in fixed source problems with the perturbation source method. Progress in Nuclear Energy, 2021, 132, 103611.	2.9	5
7	Exact Monte Carlo calculation method for $\langle i \rangle K \langle i \rangle$ -eigenvalue change using perturbation source method. Journal of Nuclear Science and Technology, 2021, 58, 886-898.	1.3	3
8	Advances in time-dependent Monte Carlo simulations for void velocity determination using neutron noise techniques. Progress in Nuclear Energy, 2021, 138, 103840.	2.9	1
9	Application of dynamic mode decomposition to exponential experiment for spatial decay constant determination. Annals of Nuclear Energy, 2021, 162, 108506.	1.8	8
10	Monte Carlo sensitivity analysis method for the effective delayed neutron fraction with the differential operator sampling method. Annals of Nuclear Energy, 2020, 140, 107108.	1.8	8
11	Convergence characteristics and Wielandt acceleration of the time source method for Monte Carlo alpha eigenvalue calculations. Annals of Nuclear Energy, 2020, 146, 107627.	1.8	2
12	Subcriticality - from basics to applications (8). Atomos, 2020, 62, 285-289.	0.0	0
13	Experimental study on local interfacial parameters in upward air-water bubbly flow in a vertical $6\hat{\text{A}}\text{--}\hat{\text{A}}\text{6}$ rod bundle. International Journal of Heat and Mass Transfer, 2019, 144, 118696.	4.8	15
14	Decomposition of neutron noise in a reactor into higher-order mode components and investigation of the space and frequency dependence. Progress in Nuclear Energy, 2019, 117, 103098.	2.9	5
15	Two-step Monte Carlo sensitivity analysis of alpha- and gamma-eigenvalues with the differential operator sampling method. Annals of Nuclear Energy, 2019, 133, 100-109.	1.8	9
16	Calculation of the cross and auto power spectral densities for low neutron counting from pulse mode detectors. Annals of Nuclear Energy, 2019, 131, 138-147.	1.8	5
17	A Monte Carlo technique for sensitivity analysis of alpha-eigenvalue with the differential operator sampling method. Annals of Nuclear Energy, 2019, 127, 178-187.	1.8	11
18	Implementation of a frequency-domain neutron noise analysis method in a production-level continuous energy Monte Carlo code: Verification and application in a BWR. Annals of Nuclear Energy, 2018, 115, 494-501.	1.8	17

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19	Eigenvalue sensitivity analysis capabilities with the differential operator method in the superhistory Monte Carlo method. <i>Annals of Nuclear Energy</i> , 2018, 112, 150-157.	1.8	13
20	Monte Carlo perturbation methods using "virtual density" theory for calculating reactivity due to geometry change. <i>Annals of Nuclear Energy</i> , 2018, 119, 362-373.	1.8	3
21	Effect of higher harmonics in the area-ratio pulsed neutron source technique. <i>Progress in Nuclear Energy</i> , 2018, 108, 286-294.	2.9	3
22	Monte Carlo method for solving a B1 equation with complex-valued buckling in asymmetric geometries and generation of directional diffusion coefficients. <i>Annals of Nuclear Energy</i> , 2018, 122, 37-46.	1.8	3
23	Improvement and performance evaluation of the perturbation source method for an exact Monte Carlo perturbation calculation in fixed source problems. <i>Journal of Computational Physics</i> , 2017, 345, 245-259.	3.8	5
24	Time-dependent Monte Carlo simulations for neutron noise in void-containing water flow. <i>Progress in Nuclear Energy</i> , 2017, 101, 270-278.	2.9	2
25	New findings on neutron noise propagation properties in void containing water using neutron noise transport calculations. <i>Progress in Nuclear Energy</i> , 2016, 90, 58-68.	2.9	7
26	Frequency domain optical tomography using a Monte Carlo perturbation method. <i>Optics Communications</i> , 2016, 364, 165-176.	2.1	6
27	Dynamic Monte Carlo calculation method by solving frequency domain transport equation using the complex-valued weight Monte Carlo method. <i>Annals of Nuclear Energy</i> , 2015, 85, 426-433.	1.8	9
28	Higher harmonic analyses of the 252 Cf source driven noise analysis method. <i>Annals of Nuclear Energy</i> , 2015, 76, 521-529.	1.8	6
29	Theory of Power Spectral Density and Feynman-Alpha Method in Accelerator-Driven System and Their Higher-Order Mode Effects. , 2015, , 119-128.		0
30	Frequency domain Monte Carlo simulation method for cross power spectral density driven by periodically pulsed spallation neutron source using complex-valued weight Monte Carlo. <i>Annals of Nuclear Energy</i> , 2014, 63, 711-720.	1.8	10
31	Higher order mode analyses of power spectral density and Feynman- α method in accelerator driven system with periodically pulsed spallation neutron source. <i>Annals of Nuclear Energy</i> , 2014, 66, 63-73.	1.8	9
32	A new concept of Monte Carlo kinetics parameter calculation using complex-valued perturbation. <i>Annals of Nuclear Energy</i> , 2014, 71, 480-488.	1.8	8
33	Improvements and New Findings in Monte Carlo Method with Complex-valued Weights for Neutron Leakage-corrected Assembly Calculations. , 2014, , .		1
34	Energy-higher order mode analyses in Feynman- α method. <i>Annals of Nuclear Energy</i> , 2013, 57, 84-91.	1.8	8
35	Monte Carlo method with complex-valued weights for frequency domain analyses of neutron noise. <i>Annals of Nuclear Energy</i> , 2013, 58, 72-79.	1.8	32
36	Monte Carlo method with complex weights for neutron leakage-corrected calculations and anisotropic diffusion coefficient generations. <i>Annals of Nuclear Energy</i> , 2012, 50, 141-149.	1.8	25

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37	Monte Carlo algorithm for buckling search and neutron leakage-corrected calculations. Annals of Nuclear Energy, 2012, 47, 14-20.	1.8	15
38	Non-regionwise weight cancellation for Monte Carlo higher order criticality calculations using kernel density estimator. Annals of Nuclear Energy, 2011, 38, 2515-2520.	1.8	10
39	Applicability of non-analog Monte Carlo technique to reactor noise simulation. Annals of Nuclear Energy, 2011, 38, 647-655.	1.8	10
40	Higher order mode analyses in Feynman- $\hat{\pm}$ method. Annals of Nuclear Energy, 2011, 38, 1231-1237.	1.8	12
41	Higher Order $\hat{\pm}$ Mode Eigenvalue Calculation by Monte Carlo Power Iteration. Progress in Nuclear Science and Technology, 2011, 2, 826-835.	0.3	26
42	Convergence of the second eigenfunction in Monte Carlo power iteration. Annals of Nuclear Energy, 2009, 36, 7-14.	1.8	19
43	Reliable Method for Fission Source Convergence of Monte Carlo Criticality Calculation with Wielandt's Method. Journal of Nuclear Science and Technology, 2004, 41, 99-107.	1.3	35
44	Real Time β Value Measurement with Feynman- $\hat{\pm}$ Method Utilizing Time Series Data Acquisition on Low Enriched Uranium System. Journal of Nuclear Science and Technology, 2004, 41, 177-182.	1.3	13
45	Real Time β Value Measurement with Feynman- β Method Utilizing Time Series Data Acquisition on Low Enriched Uranium System. Journal of Nuclear Science and Technology, 2004, 41, 177-182.	1.3	5
46	Reliable Method for Fission Source Convergence of Monte Carlo Criticality Calculation with Wielandt's Method. Journal of Nuclear Science and Technology, 2004, 41, 99-107.	1.3	1
47	Effect of Higher-Harmonic Flux in Exponential Experiment for Subcriticality Measurement. Journal of Nuclear Science and Technology, 2003, 40, 77-83.	1.3	7
48	Criticality Safety Benchmark Experiment on 10% Enriched Uranyl Nitrate Solution Using a 28-cm-Thickness Slab Core. Journal of Nuclear Science and Technology, 2002, 39, 789-799.	1.3	5
49	Reactivity Measurements of Erbium at Tank-type Critical Assembly and Analyses Using ENDF/B-VI, JEF-2.2 and preliminary JENDL-3.3 Libraries. Journal of Nuclear Science and Technology, 2002, 39, 959-962.	1.3	0
50	Mechanisms of Positive Temperature Reactivity Coefficients of Dilute Plutonium Solutions. Nuclear Science and Engineering, 2002, 142, 305-314.	1.1	2
51	Kinetic Parameter β_{eff} Measurement on Low Enriched Uranyl Nitrate Solution with Single Unit Cores (600PHI., 280T, 800PHI.) of STACY. Journal of Nuclear Science and Technology, 2002, 39, 1227-1236.	1.3	20
52	Benchmark Evaluation on Single Core System Composed of 10% Enriched Uranyl Nitrate Solution at STACY. Journal of Nuclear Science and Technology, 2002, 39, 971-973.	1.3	0
53	Modified Quasi-Steady-State Method to Evaluate the Mean Power Profiles of Nuclear Excursions in Fissile Solution. Journal of Nuclear Science and Technology, 2002, 39, 1162-1168.	1.3	6
54	Kinetic Parameter β_{eff} Measurement on Low Enriched Uranyl Nitrate Solution with Single Unit Cores (600PHI., 280T, 800PHI.) of STACY.. Journal of Nuclear Science and Technology, 2002, 39, 1227-1236.	1.3	13

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55	Fission Source Convergence of Monte Carlo Criticality Calculations in Weakly Coupled Fissile Arrays. Journal of Nuclear Science and Technology, 2000, 37, 41-52.	1.3	20
56	Fission Source Convergence of Monte Carlo Criticality Calculations in Weakly Coupled Fissile Arrays.. Journal of Nuclear Science and Technology, 2000, 37, 41-52.	1.3	5
57	Improvement of Neutron Source Introduction Method for Absolute Measurements of Low Reactor Power. Journal of Nuclear Science and Technology, 1999, 36, 1069-1075.	1.3	2
58	Perturbation of General Boundary Condition for an Eigenvalue Change in the Neutron Boltzmann Transport Equation. Nuclear Science and Engineering, 1997, 125, 19-23.	1.1	2
59	Benchmark Model of Critical Experiment at TCA for Integral Evaluation of Fission Product Nuclide Cross Sections. Journal of Nuclear Science and Technology, 1997, 34, 202-210.	1.3	4
60	Accurate Estimation of Subcriticality Using Indirect Bias Estimation Method, (II). Journal of Nuclear Science and Technology, 1997, 34, 544-550.	1.3	1
61	Measurements and Analyses of Reactivity Effect of Fission Product Nuclides in Epithermal Energy Range. Journal of Nuclear Science and Technology, 1997, 34, 1178-1184.	1.3	0
62	Accurate Estimation of Subcriticality Using Indirect Bias Estimation Method, (I). Journal of Nuclear Science and Technology, 1997, 34, 454-460.	1.3	2
63	Most Critical Geometry of a Fuel Solution Based on the Transport Boundary Perturbation Theory. Journal of Nuclear Science and Technology, 1996, 33, 78-82.	1.3	1
64	Measurements and Analyses of the Ratio of ²³⁸ U Captures to ²³⁵ U Fission in Low-Enriched UO ₂ Tight Lattices. Journal of Nuclear Science and Technology, 1994, 31, 1160-1170.	1.3	6
65	Measurements and Analyses of the Ratio of ²³⁸ U Captures to ²³⁵ U Fission in Low Enriched UO ₂ Tight Lattices.. Journal of Nuclear Science and Technology, 1994, 31, 1160-1170.	1.3	4
66	Temperature Effects on Reactivity in Light Water Moderated UO ₂ Cores with Soluble Poisons. Journal of Nuclear Science and Technology, 1992, 29, 1201-1211.	1.3	5
67	Temperature Effects on Reactivity in Light Water Moderated UO ₂ Cores with Soluble Poisons.. Journal of Nuclear Science and Technology, 1992, 29, 1201-1211.	1.3	1