Toshihiro Yamamoto

List of Publications by Year
in descending order

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67	502	759233 12	839539 18
papers	citations	h-index	g-index

67	67	67	153
all docs	docs citations	times ranked	citing authors

#	Article	IF	Citations
1	Reliable Method for Fission Source Convergence of Monte Carlo Criticality Calculation with Wielandt's Method. Journal of Nuclear Science and Technology, 2004, 41, 99-107.	1.3	35
2	Monte Carlo method with complex-valued weights for frequency domain analyses of neutron noise. Annals of Nuclear Energy, 2013, 58, 72-79.	1.8	32
3	Higher Order Î±Mode Eigenvalue Calculation by Monte Carlo Power Iteration. Progress in Nuclear Science and Technology, 2011, 2, 826-835.	0.3	26
4	Monte Carlo method with complex weights for neutron leakage-corrected calculations and anisotropic diffusion coefficient generations. Annals of Nuclear Energy, 2012, 50, 141-149.	1.8	25
5	Fission Source Convergence of Monte Carlo Criticality Calculations in Weakly Coupled Fissile Arrays. Journal of Nuclear Science and Technology, 2000, 37, 41-52.	1.3	20
6	Kinetic Parameter Î²_{eff}/â„“ Measurement on Low Enriched Uranyl Nitrate Solution with Single Unit Cores (600?;, 280T, 800?;)of STACY. Journal of Nuclear Science and Technology, 2002, 39, 1227-1236.	1.3	20
7	Convergence of the second eigenfunction in Monte Carlo power iteration. Annals of Nuclear Energy, 2009, 36, 7-14.	1.8	19
8	Implementation of a frequency-domain neutron noise analysis method in a production-level continuous energy Monte Carlo code: Verification and application in a BWR. Annals of Nuclear Energy, 2018, 115, 494-501.	1.8	17
9	Monte Carlo algorithm for buckling search and neutron leakage-corrected calculations. Annals of Nuclear Energy, 2012, 47, 14-20.	1.8	15
10	Experimental study on local interfacial parameters in upward air-water bubbly flow in a vertical 6â€Ã—â€6 rod bundle. International Journal of Heat and Mass Transfer, 2019, 144, 118696.	4.8	15
11	Real Time a Value Measurement with Feynman-Î± Method Utilizing Time Series Data Acquisition on Low Enriched Uranium System. Journal of Nuclear Science and Technology, 2004, 41, 177-182.	1.3	13
12	Eigenvalue sensitivity analysis capabilities with the differential operator method in the superhistory Monte Carlo method. Annals of Nuclear Energy, 2018, 112, 150-157.	1.8	13
13	Kinetic Parameter .BETA.eff/l Measurement on Low Enriched Uranyl Nitrate Solution with Single Unit Cores (600.PHI., 280T, 800.PHI.) of STACY.. Journal of Nuclear Science and Technology, 2002, 39, 1227-1236.	1.3	13
14	Higher order mode analyses in Feynman-Î± method. Annals of Nuclear Energy, 2011, 38, 1231-1237.	1.8	12
15	A Monte Carlo technique for sensitivity analysis of alpha-eigenvalue with the differential operator sampling method. Annals of Nuclear Energy, 2019, 127, 178-187.	1.8	11
16	Non-regionwise weight cancellation for Monte Carlo higher order criticality calculations using kernel density estimator. Annals of Nuclear Energy, 2011, 38, 2515-2520.	1.8	10
17	Applicability of non-analog Monte Carlo technique to reactor noise simulation. Annals of Nuclear Energy, 2011, 38, 647-655.	1.8	10
18	Frequency domain Monte Carlo simulation method for cross power spectral density driven by periodically pulsed spallation neutron source using complex-valued weight Monte Carlo. Annals of Nuclear Energy, 2014, 63, 711-720.	1.8	10

#	Article	IF	Citations
19	Higher order mode analyses of power spectral density and Feynman-Î± method in accelerator driven system with periodically pulsed spallation neutron source. Annals of Nuclear Energy, 2014, 66, 63-73.	1.8	9
20	Dynamic Monte Carlo calculation method by solving frequency domain transport equation using the complex-valued weight Monte Carlo method. Annals of Nuclear Energy, 2015, 85, 426-433.	1.8	9
21	Two-step Monte Carlo sensitivity analysis of alpha- and gamma-eigenvalues with the differential operator sampling method. Annals of Nuclear Energy, 2019, 133, 100-109.	1.8	9
22	Energy-higher order mode analyses in Feynman-Î± method. Annals of Nuclear Energy, 2013, 57, 84-91.	1.8	8
23	A new concept of Monte Carlo kinetics parameter calculation using complex-valued perturbation. Annals of Nuclear Energy, 2014, 71, 480-488.	1.8	8
24	Monte Carlo sensitivity analysis method for the effective delayed neutron fraction with the differential operator sampling method. Annals of Nuclear Energy, 2020, 140, 107108.	1.8	8
25	Application of dynamic mode decomposition to exponential experiment for spatial decay constant determination. Annals of Nuclear Energy, 2021, 162, 108506.	1.8	8
26	Effect of Higher-Harmonic Flux in Exponential Experiment for Subcriticality Measurement. Journal of Nuclear Science and Technology, 2003, 40, 77-83.	1.3	7
27	New findings on neutron noise propagation properties in void containing water using neutron noise transport calculations. Progress in Nuclear Energy, 2016, 90, 58-68.	2.9	7
28	Measurements and Analyses of the Ratio of238U Captures to235U Fission in Low-Enriched UO2Tight Lattices. Journal of Nuclear Science and Technology, 1994, 31, 1160-1170.	1.3	6
29	Modified Quasi-Steady-State Method to Evaluate the Mean Power Profiles of Nuclear Excursions in Fissile Solution. Journal of Nuclear Science and Technology, 2002, 39, 1162-1168.	1.3	6
30	Higher harmonic analyses of the 252 Cf source driven noise analysis method. Annals of Nuclear Energy, 2015, 76, 521-529.	1.8	6
31	Frequency domain optical tomography using a Monte Carlo perturbation method. Optics Communications, 2016, 364, 165-176.	2.1	6
32	Temperature Effects on Reactivity in Light Water Moderated U02Cores with Soluble Poisons. Journal of Nuclear Science and Technology, 1992, 29, 1201-1211.	1.3	5
33	Criticality Safety Benchmark Experiment on 10% Enriched Uranyl Nitrate Solution Using a 28-cm-Thickness Slab Core. Journal of Nuclear Science and Technology, 2002, 39, 789-799.	1.3	5
34	Improvement and performance evaluation of the perturbation source method for an exact Monte Carlo perturbation calculation in fixed source problems. Journal of Computational Physics, 2017, 345, 245-259.	3.8	5
35	Decomposition of neutron noise in a reactor into higher-order mode components and investigation of the space and frequency dependence. Progress in Nuclear Energy, 2019, 117, 103098.	2.9	5
36	Calculation of the cross and auto power spectral densities for low neutron counting from pulse mode detectors. Annals of Nuclear Energy, 2019, 131, 138-147.	1.8	5

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37	Monte Carlo perturbation calculation for geometry change in fixed source problems with the perturbation source method. Progress in Nuclear Energy, 2021, 132, 103611.	2.9	5
38	Fission Source Convergence of Monte Carlo Criticality Calculations in Weakly Coupled Fissile Arrays.. Journal of Nuclear Science and Technology, 2000, 37, 41-52.	1.3	5
39	Real Time .ALPHA. Value Measurement with Feynman-.ALPHA. Method Utilizing Time Series Data Acquisition on Low Enriched Uranium System. Journal of Nuclear Science and Technology, 2004, 41, 177-182.	1.3	5
40	Benchmark Model of Critical Experiment at TCA for Integral Evaluation of Fission Product Nuclide Cross Sections. Journal of Nuclear Science and Technology, 1997, 34, 202-210.	1.3	4
41	Frequency domain Monte Carlo simulations of void velocity measurements in an actual experimental setup using a neutron noise technique. Journal of Nuclear Science and Technology, 2021, 58, 190-200.	1.3	4
42	Measurements and Analyses of the Ratio of 238U Captures to 235U Fission in Low Enriched UO2 Tight Lattices.. Journal of Nuclear Science and Technology, 1994, 31, 1160-1170.	1.3	4
43	Higher harmonic analyses of the Rossi-Î± method and application of dynamic mode decomposition for time decay constant determination in a 1D subcritical system. Annals of Nuclear Energy, 2022, 168, 108886.	1.8	4
44	Monte Carlo perturbation methods using â€œvirtual densityâ€•theory for calculating reactivity due to geometry change. Annals of Nuclear Energy, 2018, 119, 362-373.	1.8	3
45	Effect of higher harmonics in the area-ratio pulsed neutron source technique. Progress in Nuclear Energy, 2018, 108, 286-294.	2.9	3
46	Monte Carlo method for solving a B1 equation with complex-valued buckling in asymmetric geometries and generation of directional diffusion coefficients. Annals of Nuclear Energy, 2018, 122, 37-46.	1.8	3
47	Exact Monte Carlo calculation method for K-eigenvalue change using perturbation source method. Journal of Nuclear Science and Technology, 2021, 58, 886-898.	1.3	3
48	Perturbation of General Boundary Condition for an Eigenvalue Change in the Neutron Boltzmann Transport Equation. Nuclear Science and Engineering, 1997, 125, 19-23.	1.1	2
49	Improvement of Neutron Source Introduction Method for Absolute Measurements of Low Reactor Power. Journal of Nuclear Science and Technology, 1999, 36, 1069-1075.	1.3	2
50	Mechanisms of Positive Temperature Reactivity Coefficients of Dilute Plutonium Solutions. Nuclear Science and Engineering, 2002, 142, 305-314.	1.1	2
51	Time-dependent Monte Carlo simulations for neutron noise in void-containing water flow. Progress in Nuclear Energy, 2017, 101, 270-278.	2.9	2
52	Convergence characteristics and Wielandt acceleration of the time source method for Monte Carlo alpha eigenvalue calculations. Annals of Nuclear Energy, 2020, 146, 107627.	1.8	2
53	Accurate Estimation of Subcriticality Using Indirect Bias Estimation Method, (I). Journal of Nuclear Science and Technology, 1997, 34, 454-460.	1.3	2
54	Monte Carlo sensitivity calculation in fixed source problems with the derivative source method. Journal of Computational Physics, 2022, 460, 111155.	3.8	2

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55	Most Critical Geometry of a Fuel Solution Based on the Transport Boundary Perturbation Theory. Journal of Nuclear Science and Technology, 1996, 33, 78-82.	1.3	1
56	Accurate Estimation of Subcriticality Using Indirect Bias Estimation Method, (II). Journal of Nuclear Science and Technology, 1997, 34, 544-550.	1.3	1
57	Advances in time-dependent Monte Carlo simulations for void velocity determination using neutron noise techniques. Progress in Nuclear Energy, 2021, 138, 103840.	2.9	1
58	Improvements and New Findings in Monte Carlo Method with Complex-valued Weights for Neutron Leakage-corrected Assembly Calculations. , 2014, , .		1
59	Temperature Effects on Reactivity in Light Water Moderated UO2 Cores with Soluble Poisons.. Journal of Nuclear Science and Technology, 1992, 29, 1201-1211.	1.3	1
60	Reliable Method for Fission Source Convergence of Monte Carlo Criticality Calculation with Wielandt's Method. Journal of Nuclear Science and Technology, 2004, 41, 99-107.	1.3	1
61	Dynamic mode decomposition application to dominance ratio assessment in Monte Carlo k-eigenvalue calculation. Annals of Nuclear Energy, 2022, 175, 109205.	1.8	1
62	Measurements and Analyses of Reactivity Effect of Fission Product Nuclides in Epithermal Energy Range. Journal of Nuclear Science and Technology, 1997, 34, 1178-1184.	1.3	0
63	Reactivity Measurements of Erbium at Tank-type Critical Assembly and Analyses Using ENDF/B-VI, JEF-2.2 and preliminary JENDL-3.3 Libraries. Journal of Nuclear Science and Technology, 2002, 39, 959-962.	1.3	0
64	Benchmark Evaluation on Single Core System Composed of 10% Enriched Uranyl Nitrate Solution at STACY. Journal of Nuclear Science and Technology, 2002, 39, 971-973.	1.3	0
65	Theory of Power Spectral Density and Feynman-Alpha Method in Accelerator-Driven System and Their Higher-Order Mode Effects. , 2015, , 119-128.		0
66	Subcriticality - from basics to applications (8). Atomos, 2020, 62, 285-289.	0.0	0
67	Deterministic and stochastic methods for sensitivity analysis of neutron noise. Progress in Nuclear Energy, 2022, 145, 104130.	2.9	0