

Piotr Darnowski

List of Publications by Year in descending order

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Version: 2024-02-01

24
papers

93
citations

1478505

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1588992

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25
all docs

25
docs citations

25
times ranked

54
citing authors

#	ARTICLE	IF	CITATIONS
1	Investigation of the recriticality potential during reflooding phase of Fukushima Daiichi Unit-3 accident. <i>Annals of Nuclear Energy</i> , 2017, 99, 495-509.	1.8	10
2	Demonstration of the E-BEPU methodology for SL-LOCA in a Gen-III PWR reactor. <i>Reliability Engineering and System Safety</i> , 2022, 226, 108707.	8.9	10
3	Analysis of the BEAVRS PWR benchmark using SCALE and PARCS. <i>Nukleonika</i> , 2019, 64, 87-96.	0.8	8
4	Total loss of AC power analysis for EPR reactor. <i>Nuclear Engineering and Design</i> , 2015, 289, 8-18.	1.7	7
5	Loss of Coolant Accident in Pressurized Water Reactor. Prediction of a 6-inch Cold Leg Break with Relap5 and Cathare 2. <i>Procedia Engineering</i> , 2016, 157, 333-340.	1.2	7
6	Study of the material release during PHÅ%BUS FPT-1 bundle phase with MELCOR 2.2. <i>Annals of Nuclear Energy</i> , 2020, 148, 107700.	1.8	6
7	Minor actinides impact on basic safety parameters of medium-sized sodium-cooled fast reactor. <i>Nukleonika</i> , 2015, 60, 171-179.	0.8	5
8	Global uncertainty and sensitivity analysis of MELCOR and TRACE critical flow models against MARVIKEN tests. <i>Nuclear Engineering and Design</i> , 2021, 378, 111150.	1.7	5
9	Uncertainty and Sensitivity Analysis of the In-Vessel Hydrogen Generation for Gen-III PWR and Phebus FPT-1 with MELCOR 2.2. <i>Energies</i> , 2021, 14, 4884.	3.1	5
10	Preliminary Analysis of an Aged RPV Subjected to Station Blackout. <i>Energies</i> , 2021, 14, 4394.	3.1	4
11	Monte Carlo Analysis of the Battery-Type High Temperature Gas Cooled Reactor. <i>Archives of Thermodynamics</i> , 2017, 38, 209-227.	1.0	4
12	Pre-Conceptual Design of the Research High-Temperature Gas-Cooled Reactor TeResa for Non-Electrical Applications. <i>Energies</i> , 2022, 15, 2084.	3.1	4
13	The development of a novel adaptive genetic algorithm for the optimization of fuel cycle length. <i>Annals of Nuclear Energy</i> , 2021, 155, 108153.	1.8	3
14	Solid-fuel Rocket Engines: Layered Composite Materials Manufacturing and Thermal Diffusivity Measurements. <i>High Temperature Materials and Processes</i> , 2014, 33, 171-177.	1.4	2
15	Monte Carlo simulations of the 1000 MWth SFR OECD/NEA benchmark with SERPENT code. <i>AIP Conference Proceedings</i> , 2019, , .	0.4	2
16	Neutronic performance of a thorium based mixed oxide fuel in a burner sodium-cooled fast reactor. <i>Energy</i> , 2020, 212, 118744.	8.8	2
17	Optimization of the loading pattern of the PWR core using genetic algorithms and multi-purpose fitness function. <i>Nukleonika</i> , 2021, 66, 147-151.	0.8	2
18	Application of genetic algorithms in optimization of SFR nuclear reactor design. <i>Nukleonika</i> , 2021, 66, 139-145.	0.8	2

#	ARTICLE	IF	CITATIONS
19	Development of One-way-coupling Methodology between Severe Accident Integral Code MELCOR and Monte Carlo Neutron Transport Code SERPENT. Procedia Engineering, 2016, 157, 207-213.	1.2	1
20	A Sensitivity Study of Critical Flow Modeling with MELCOR 2.2 Code Based on the Marviken CFT-21 Experiment. Energies, 2021, 14, 4985.	3.1	1
21	Coordination number for random distribution of parallel fibres. Archives of Thermodynamics, 2017, 38, 3-26.	1.0	1
22	Analysis of effects of formation of non-condensable gases and water vapor during a severe accident in a boiling water nuclear reactor Analiza skutków powstawania niekondensujących gazów i pary wodnej podczas ciężkiej awarii wrzącego reaktora jądrowego. Przemysł Chemiczny, 2017, 1, 51-54.	0.0	1
23	Thorium application in the medium-sized sodium-cooled fast reactor. E3S Web of Conferences, 2019, 137, 01030.	0.5	0
24	Study of the Ageing Effects on the Lower Head Failure in a PWR Reactor. , 2020, , .		0