

Tim D Burchell

List of Publications by Citations

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The third column is the impact factor (IF) of the journal, and the fourth column is the number of citations of the article.

48
papers

1,640
citations

22
h-index

40
g-index

48
ext. papers

1,787
ext. citations

5.8
avg, IF

4.43
L-index

#	Paper	IF	Citations
48	High-thermal-conductivity, mesophase-pitch-derived carbon foams: effect of precursor on structure and properties. <i>Carbon</i> , 2000 , 38, 953-973	10.4	360
47	A novel process and material for the separation of carbon dioxide and hydrogen sulfide gas mixtures. <i>Carbon</i> , 1997 , 35, 1279-1294	10.4	114
46	A microstructurally based fracture model for polygranular graphites. <i>Carbon</i> , 1996 , 34, 297-316	10.4	88
45	Effect of temperature on the adsorption of water in porous carbons. <i>Langmuir</i> , 2005 , 21, 9457-67	4	78
44	Swelling of nuclear graphite and high quality carbon fiber composite under very high irradiation temperature. <i>Journal of Nuclear Materials</i> , 2008 , 381, 55-61	3.3	71
43	The effect of high fluence neutron irradiation on the properties of a fine-grained isotropic nuclear graphite. <i>Journal of Nuclear Materials</i> , 1996 , 230, 1-7	3.3	66
42	The effect of microstructure on air oxidation resistance of nuclear graphite. <i>Carbon</i> , 2012 , 50, 3354-3366	10.4	65
41	Effects of heat treatment conditions on the thermal properties of mesophase pitch-derived graphitic foams. <i>Carbon</i> , 2004 , 42, 1849-1852	10.4	62
40	Practical aspects for characterizing air oxidation of graphite. <i>Journal of Nuclear Materials</i> , 2008 , 381, 15-24	3.3	61
39	The effect of neutron irradiation damage on the properties of grade NBG-10 graphite. <i>Journal of Nuclear Materials</i> , 2007 , 371, 18-27	3.3	58
38	Thermal conductivity degradation of graphites due to neutron irradiation at low temperature. <i>Journal of Nuclear Materials</i> , 1995 , 224, 222-229	3.3	43
37	Structure-related property changes in polycrystalline graphite under neutron irradiation. <i>Carbon</i> , 1994 , 32, 499-505	10.4	43
36	Pore structure development in oxidized IG-110 nuclear graphite. <i>Journal of Nuclear Materials</i> , 2012 , 430, 229-238	3.3	39
35	A novel approach to fabricating fuel compacts for the next generation nuclear plant (NGNP). <i>Journal of Nuclear Materials</i> , 2008 , 381, 25-38	3.3	35
34	Irradiation induced creep behavior of H-451 graphite. <i>Journal of Nuclear Materials</i> , 2008 , 381, 46-54	3.3	30
33	Passive CO ₂ removal using a carbon fiber composite molecular sieve. <i>Energy Conversion and Management</i> , 1996 , 37, 947-954	10.6	30
32	The fracture of polygranular graphites. <i>Carbon</i> , 1986 , 24, 581-602	10.4	30

31	A novel carbon fiber based material and separation technology. <i>Energy Conversion and Management</i> , 1997 , 38, S99-S104	10.6	28
30	Radiation damage in carbon-carbon composites: structure and property effects. <i>Physica Scripta</i> , 1996 , T64, 17-25	2.6	28
29	The analysis of irradiation creep experiments on nuclear reactor graphite. <i>Carbon</i> , 1994 , 32, 119-125	10.4	28
28	Early Damage Mechanisms in Nuclear Grade Graphite under Irradiation. <i>Materials Research Letters</i> , 2014 , 2, 43-50	7.4	27
27	Penetration depth and transient oxidation of graphite by oxygen and water vapor. <i>Journal of Nuclear Materials</i> , 2009 , 393, 518-521	3.3	22
26	Oxidation of PCEA nuclear graphite by low water concentrations in helium. <i>Journal of Nuclear Materials</i> , 2014 , 453, 225-232	3.3	20
25	The relationship between microstructure and the reduction of elastic modulus in thermally and radiolytically corroded nuclear graphites. <i>Carbon</i> , 1986 , 24, 545-549	10.4	20
24	A study of the annealing behavior of neutron irradiated graphite. <i>Carbon</i> , 2011 , 49, 3-10	10.4	18
23	Acoustic emission from polygranular graphites. <i>Carbon</i> , 1985 , 23, 739-747	10.4	18
22	Strength of neutron-irradiated high-quality 3D carbon fiber composite. <i>Journal of Nuclear Materials</i> , 2003 , 321, 165-169	3.3	16
21	Modeling the multiaxial strength of H-451 nuclear grade graphite. <i>Carbon</i> , 2007 , 45, 2570-2583	10.4	15
20	Selection of water-dispersible carbon black for fabrication of uranium oxycarbide microspheres. <i>Journal of Nuclear Materials</i> , 2008 , 375, 38-51	3.3	14
19	Irradiation effects on graphite foam. <i>Carbon</i> , 2006 , 44, 618-628	10.4	14
18	Water adsorption and desorption on microporous solids at elevated temperature. <i>Journal of Thermal Analysis and Calorimetry</i> , 2005 , 81, 609-615	4.1	13
17	The shear fracture toughness, K, of graphite. <i>Carbon</i> , 2016 , 98, 267-279	10.4	11
16	The effect of neutron irradiation on the fracture toughness of graphite. <i>Nuclear Engineering and Design</i> , 2014 , 271, 262-269	1.8	10
15	Acoustic emission from irradiated nuclear graphite. <i>Journal of Nuclear Materials</i> , 1986 , 140, 11-18	3.3	10
14	Recycling irradiated nuclear graphite: a greener path forward. <i>Nuclear Engineering and Design</i> , 2012 , 251, 69-77	1.8	7

13	Principles and practice of a bellows-loaded compact irradiation vehicle. <i>Journal of Nuclear Materials</i> , 2013 , 439, 108-116	3.3	6
12	Modeling irradiation creep of graphite using rate theory. <i>Journal of Nuclear Materials</i> , 2016 , 473, 197-205	3.3	6
11	Effect of specimen size and grain orientation on the mechanical and physical properties of NBG-18 nuclear graphite. <i>Journal of Nuclear Materials</i> , 2015 , 462, 1-7	3.3	5
10	Preliminary data on processing and characterization of recycled irradiated graphite. <i>Carbon</i> , 2010 , 48, 3303-3305	10.4	5
9	Use of Carbon Fibre Composite Molecular Sieves for Air Separation. <i>Adsorption Science and Technology</i> , 2005 , 23, 175-194	3.6	5
8	Vacuum-plasma-sprayed silicon coatings. <i>Surface and Coatings Technology</i> , 1991 , 49, 24-30	4.4	5
7	Density Change of an Oxidized Nuclear Graphite by Acoustic Microscopy and Image Processing. <i>Journal of Engineering for Gas Turbines and Power</i> , 2009 , 131,	1.7	4
6	Preliminary design of a graphite irradiation tensile creep experiment in the target region of the high flux isotope reactor. <i>Journal of Nuclear Materials</i> , 2008 , 381, 119-123	3.3	4
5	Biaxial Strength and Fracture Criterion for HTGR Graphites. <i>Journal of Nuclear Science and Technology</i> , 1997 , 34, 476-483	1	3
4	Leveraging comprehensive baseline datasets to quantify property variability in nuclear-grade graphites. <i>Nuclear Engineering and Design</i> , 2016 , 307, 77-85	1.8	3
3	Applicability and Limitations of Miniature Specimens for Properties Determination of Fine-Grained Graphite 2014 , 65-83		1
2	Experimental plan and design of two experiments for graphite irradiation at temperatures up to 1500°C in the target region of the high flux isotope reactor. <i>Journal of Nuclear Materials</i> , 2008 , 381, 114-118	3.3	1
1	Small Specimen Data from a High Temperature HFIR Irradiation Experiment 2014 , 172-185		0