

Kan Wang

List of Publications by Year in descending order

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150
papers

1,301
citations

471509

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477307

29
g-index

150
all docs

150
docs citations

150
times ranked

446
citing authors

| # | ARTICLE | IF | CITATIONS |
|----|---|-----|-----------|
| 1 | RMC "A Monte Carlo code for reactor core analysis. Annals of Nuclear Energy, 2015, 82, 121-129. | 1.8 | 215 |
| 2 | Development of burnup methods and capabilities in Monte Carlo code RMC. Annals of Nuclear Energy, 2013, 51, 289-294. | 1.8 | 44 |
| 3 | A new power mapping method based on ordinary kriging and determination of optimal detector location strategy. Annals of Nuclear Energy, 2014, 68, 118-123. | 1.8 | 43 |
| 4 | Analysis of BEAVRS two-cycle benchmark using RMC based on full core detailed model. Progress in Nuclear Energy, 2017, 98, 301-312. | 2.9 | 40 |
| 5 | BEAVRS full core burnup calculation in hot full power condition by RMC code. Annals of Nuclear Energy, 2017, 101, 434-446. | 1.8 | 35 |
| 6 | Coupled neutronics/thermal-hydraulics analysis of a full PWR core using RMC and CTF. Annals of Nuclear Energy, 2017, 109, 327-336. | 1.8 | 34 |
| 7 | Development of on-the-fly temperature-dependent cross-sections treatment in RMC code. Annals of Nuclear Energy, 2016, 94, 144-149. | 1.8 | 32 |
| 8 | Development of the point-depletion code DEPTH. Nuclear Engineering and Design, 2013, 258, 235-240. | 1.7 | 30 |
| 9 | New strategies of sensitivity analysis capabilities in continuous-energy Monte Carlo code RMC. Annals of Nuclear Energy, 2015, 81, 50-61. | 1.8 | 28 |
| 10 | Coupling of RMC and CFX for analysis of Pebble Bed-Advanced High Temperature Reactor core. Nuclear Engineering and Design, 2012, 250, 385-391. | 1.7 | 27 |
| 11 | Asymptotic Wielandt Method and Superhistory Method for Source Convergence in Monte Carlo Criticality Calculation. Nuclear Science and Engineering, 2012, 172, 127-137. | 1.1 | 25 |
| 12 | Development of random geometry capability in RMC code for stochastic media analysis. Annals of Nuclear Energy, 2015, 85, 903-908. | 1.8 | 25 |
| 13 | Computing eigenvalue sensitivity coefficients to nuclear data by adjoint superhistory method and adjoint Wielandt method implemented in RMC code. Annals of Nuclear Energy, 2016, 87, 228-241. | 1.8 | 25 |
| 14 | Development of sensitivity analysis capabilities of generalized responses to nuclear data in Monte Carlo code RMC. Annals of Nuclear Energy, 2016, 97, 142-152. | 1.8 | 24 |
| 15 | RMC/CTF multiphysics solutions to VERA core physics benchmark problem 9. Annals of Nuclear Energy, 2019, 133, 837-852. | 1.8 | 23 |
| 16 | Astrophysical $\langle \text{mml:math} \text{xmlns:mml="http://www.w3.org/1998/Math/MathML"} \rangle \langle \text{mml:mi} \rangle S \langle \text{mml:mi} \rangle \langle \text{mml:math} \rangle$ factor of the $\langle \text{mml:math} \text{xmlns:mml="http://www.w3.org/1998/Math/MathML"} \rangle \langle \text{mml:mrow} \rangle \langle \text{mml:mmultiscripts} \rangle \langle \text{mml:mtext} \rangle C \langle \text{mml:mtext} \rangle \langle \text{mml:mprescript} \rangle \langle \text{mml:none} \rangle \langle \text{mml:mn} \rangle 12 \langle \text{mml:mn} \rangle \langle \text{mml:mmultiscripts} \rangle \langle \text{mml:mo} \rangle \langle \text{mml:mi} \rangle \hat{\pm} \langle \text{mml:mi} \rangle \langle \text{mml:mo} \rangle \langle \text{mml:mi} \rangle \hat{3} \langle \text{mml:mn} \rangle 18$ | 2.9 | 18 |
| 17 | A new neutronics-thermal-mechanics multi-physics coupling method for heat pipe cooled reactor based on RMC and OpenFOAM. Progress in Nuclear Energy, 2021, 139, 103842. | 2.9 | 18 |
| 18 | 2D full-core Monte Carlo pin-by-pin burnup calculations with the RMC code. Annals of Nuclear Energy, 2014, 64, 201-205. | 1.8 | 17 |

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| 19 | Accelerating a three-dimensional MOC calculation using GPU with CUDA and two-level GCMFD method. <i>Annals of Nuclear Energy</i> , 2013, 62, 445-451. | 1.8 | 15 |
| 20 | Heat Transfer Calculation on Plate-Type Fuel Assembly of High Flux Research Reactor. <i>Science and Technology of Nuclear Installations</i> , 2015, 2015, 1-13. | 0.8 | 14 |
| 21 | Computing eigenvalue sensitivity coefficients to nuclear data based on the CLUTCH method with RMC code. <i>Annals of Nuclear Energy</i> , 2016, 88, 237-251. | 1.8 | 14 |
| 22 | Improved adaptive variance reduction algorithm based on RMC code for deep penetration problems. <i>Annals of Nuclear Energy</i> , 2020, 137, 107113. | 1.8 | 14 |
| 23 | Single-step Monte Carlo criticality algorithm. <i>Computer Physics Communications</i> , 2022, 279, 108439. | 7.5 | 14 |
| 24 | Development of three methods for control rod position monitoring based on fixed in-core neutron detectors. <i>Annals of Nuclear Energy</i> , 2015, 79, 78-86. | 1.8 | 13 |
| 25 | Random geometry capability in RMC code for explicit analysis of polytype particle/pebble and applications to HTR-10 benchmark. <i>Annals of Nuclear Energy</i> , 2018, 111, 41-49. | 1.8 | 13 |
| 26 | One-step Monte Carlo global homogenization based on RMC code. <i>Nuclear Engineering and Technology</i> , 2019, 51, 1209-1217. | 2.3 | 13 |
| 27 | Control rod position reconstruction based on K-Nearest Neighbor Method. <i>Annals of Nuclear Energy</i> , 2017, 102, 231-235. | 1.8 | 12 |
| 28 | A new nonlinear iterative method for SPN theory. <i>Annals of Nuclear Energy</i> , 2017, 110, 920-927. | 1.8 | 12 |
| 29 | Neutronics and thermal-hydraulics coupling analysis in accelerator-driven subcritical system. <i>Progress in Nuclear Energy</i> , 2020, 122, 103235. | 2.9 | 12 |
| 30 | Numerical investigation on the startup performance of high-temperature heat pipes for heat pipe cooled reactor application. <i>Nuclear Science and Techniques/Hewuli</i> , 2021, 32, 1. | 3.4 | 12 |
| 31 | A numerical solution to the nonlinear point kinetics equations using Magnus expansion. <i>Annals of Nuclear Energy</i> , 2016, 89, 84-89. | 1.8 | 11 |
| 32 | An adaptive variance reduction algorithm based on RMC code for solving deep penetration problems. <i>Annals of Nuclear Energy</i> , 2019, 128, 171-180. | 1.8 | 11 |
| 33 | RMC - A Monte Carlo Code for Reactor Core Analysis. , 2014, , . | | 11 |
| 34 | Perturbation based Monte Carlo criticality search in density, enrichment and concentration. <i>Annals of Nuclear Energy</i> , 2015, 76, 350-356. | 1.8 | 10 |
| 35 | Development of new variance reduction methods based on weight window technique in RMC code. <i>Progress in Nuclear Energy</i> , 2016, 90, 197-203. | 2.9 | 10 |
| 36 | Whole-core forward-adjoint neutron transport solutions with coupled 2-D MOC and 1-D SN and kinetics parameter calculation. <i>Progress in Nuclear Energy</i> , 2018, 108, 310-318. | 2.9 | 10 |

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| 37 | Passengersâ€™ Evacuation in Ships Based on Neighborhood Particle Swarm Optimization. <i>Mathematical Problems in Engineering</i> , 2014, 2014, 1-10. | 1.1 | 9 |
| 38 | Core axial power shape reconstruction based on radial basis function neural network. <i>Annals of Nuclear Energy</i> , 2014, 73, 339-344. | 1.8 | 9 |
| 39 | Fault detection and isolation for self powered neutron detectors based on Principal Component Analysis. <i>Annals of Nuclear Energy</i> , 2015, 85, 213-219. | 1.8 | 9 |
| 40 | Neutronics comparative analysis of plate-type research reactor using deterministic and stochastic methods. <i>Annals of Nuclear Energy</i> , 2015, 79, 133-142. | 1.8 | 9 |
| 41 | Reaction rate tally and depletion calculation with on-the-fly temperature treatment. <i>Annals of Nuclear Energy</i> , 2016, 92, 277-283. | 1.8 | 9 |
| 42 | Calculation of adjoint-weighted kinetic parameters with the reactor Monte Carlo code RMC. <i>Progress in Nuclear Energy</i> , 2017, 101, 424-434. | 2.9 | 9 |
| 43 | Eigenvalue sensitivity and uncertainty analysis based on a 2-D/1-D whole-core transport code KYADJ. <i>Annals of Nuclear Energy</i> , 2018, 122, 185-192. | 1.8 | 9 |
| 44 | Kinetic methods in Monte Carlo code RMC and its implementation to C5G7-TD benchmark. <i>Annals of Nuclear Energy</i> , 2021, 151, 107864. | 1.8 | 9 |
| 45 | SP3-coupled global variance reduction method based on RMC code. <i>Nuclear Science and Techniques/Hewuli</i> , 2021, 32, 1. | 3.4 | 9 |
| 46 | Using Generalized Laguerre Polynomials to Compute the Matrix Exponential in Burnup Equations. <i>Nuclear Science and Engineering</i> , 2013, 175, 259-265. | 1.1 | 8 |
| 47 | Mixed-Crossing-Avoided Escape Routing of Mixed-Pattern Signals on Staggered-Pin-Array PCBs. <i>IEEE Transactions on Computer-Aided Design of Integrated Circuits and Systems</i> , 2014, 33, 571-584. | 2.7 | 8 |
| 48 | Robust filtering for dynamic compensation of self-powered neutron detectors. <i>Nuclear Engineering and Design</i> , 2014, 280, 122-129. | 1.7 | 8 |
| 49 | Comparison of reactivity estimation performance between two extended Kalman filtering schemes. <i>Annals of Nuclear Energy</i> , 2016, 96, 76-82. | 1.8 | 8 |
| 50 | Domain Decomposition Strategy for Pin-wise Full-Core Monte Carlo Depletion Calculation with the Reactor Monte Carlo Code. <i>Nuclear Engineering and Technology</i> , 2016, 48, 635-641. | 2.3 | 8 |
| 51 | Depletion benchmarks calculation of random media using explicit modeling approach of RMC. <i>Annals of Nuclear Energy</i> , 2016, 87, 167-175. | 1.8 | 8 |
| 52 | Criticality benchmarking of ENDF/B-â€¦.0 and JEFF-3.3 neutron data libraries with RMC code. <i>Nuclear Engineering and Technology</i> , 2020, 52, 1917-1925. | 2.3 | 8 |
| 53 | Optimal Batch Size Growth for Wielandt Method and Superhistory Method. <i>Nuclear Science and Engineering</i> , 2022, 196, 183-192. | 1.1 | 8 |
| 54 | Optimization treatment of point-wise nuclear data in Monte Carlo criticality and burnup calculations. <i>Annals of Nuclear Energy</i> , 2011, 38, 1489-1495. | 1.8 | 7 |

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| 55 | Versatility and stabilization improvements of full core neutronics/thermal-hydraulics coupling between RMC and CTF. Nuclear Engineering and Design, 2018, 332, 88-98. | 1.7 | 7 |
| 56 | On-the-fly treatment of temperature dependent cross sections in the unresolved resonance region in RMC code. Annals of Nuclear Energy, 2018, 111, 234-241. | 1.8 | 7 |
| 57 | A Deep Learning approach for Modulation Recognition. , 2018, , . | | 7 |
| 58 | A direct calculation method for subcritical multiplication factor in Reactor Monte Carlo code RMC. Annals of Nuclear Energy, 2018, 118, 81-91. | 1.8 | 7 |
| 59 | Uniform variance method for accelerated Monte Carlo criticality calculation. Progress in Nuclear Energy, 2021, 139, 103858. | 2.9 | 7 |
| 60 | Rethinking thermal via planning with timing-power-temperature dependence for 3D ICs. , 2011, , . | | 6 |
| 61 | Burnup-dependent core neutronics analysis of plate-type research reactor using deterministic and stochastic methods. Annals of Nuclear Energy, 2015, 85, 830-836. | 1.8 | 6 |
| 62 | The Optimal Source Bias Method based on RMC code. Annals of Nuclear Energy, 2018, 121, 525-530. | 1.8 | 6 |
| 63 | Coupling RMC and CFD for simulation of transients in TREAT reactor. Annals of Nuclear Energy, 2019, 132, 249-257. | 1.8 | 6 |
| 64 | Superhistory-based differential operator method for generalized responses sensitivity calculations. Annals of Nuclear Energy, 2020, 140, 107291. | 1.8 | 6 |
| 65 | Thermal effects of leakage power in 3D ICs. , 2010, , . | | 5 |
| 66 | An Interval Bound Algorithm of optimizing reactor core loading pattern by using reactivity interval schema. Annals of Nuclear Energy, 2011, 38, 2787-2796. | 1.8 | 5 |
| 67 | Voltage island-driven power optimization for application specific network-on-chip design. , 2012, , . | | 5 |
| 68 | Escape routing of mixed-pattern signals based on staggered-pin-array PCBs. , 2013, , . | | 5 |
| 69 | Dynamic compensation of Vanadium self powered neutron detectors based on Luenberger form filter. Progress in Nuclear Energy, 2015, 78, 190-195. | 2.9 | 5 |
| 70 | Calculating the k-Eigenvalue Sensitivity to Typical Geometric Perturbations with the Adjoint-Weighted Method in the Continuous-Energy Reactor Monte Carlo Code RMC. Nuclear Science and Engineering, 2019, 193, 1186-1218. | 1.1 | 5 |
| 71 | Acceleration method of fission source convergence based on RMC code. Nuclear Engineering and Technology, 2020, 52, 1347-1354. | 2.3 | 5 |
| 72 | Improvement of sensitivity and uncertainty analysis capabilities of generalized response in Monte Carlo code RMC. Annals of Nuclear Energy, 2021, 154, 108099. | 1.8 | 5 |

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| 73 | Convergence diagnostics for Monte Carlo fission source distributions using the Wasserstein distance measure. Nuclear Engineering and Design, 2022, 389, 111675. | 1.7 | 5 |
| 74 | Thermal-to-fusion neutron convertor and Monte Carlo coupled simulation of deuteron/triton transport and secondary products generation. Nuclear Instruments & Methods in Physics Research B, 2012, 287, 19-25. | 1.4 | 4 |
| 75 | Undersampling diagnostics by fission matrix in Monte Carlo criticality calculations. Annals of Nuclear Energy, 2013, 62, 321-325. | 1.8 | 4 |
| 76 | A Monte Carlo code to get response spectrum of ions for Neutron Depth Profiling. Journal of Radioanalytical and Nuclear Chemistry, 2014, 301, 213-220. | 1.5 | 4 |
| 77 | The numerical solution of space-dependent neutron kinetics equations in hexagonal-z geometry using backward differentiation formula with adaptive step size. Annals of Nuclear Energy, 2019, 128, 203-208. | 1.8 | 4 |
| 78 | Research on the on-the-fly homogenization method based on RMC code for criticality calculations. Annals of Nuclear Energy, 2020, 135, 106985. | 1.8 | 4 |
| 79 | Preliminary verification of incompressible Navier-Stokes equations solved by The Newton method. International Journal of Advanced Nuclear Reactor Design and Technology, 2020, 2, 69-85. | 1.3 | 4 |
| 80 | RMC/CTF multiphysics solutions to VERA core physics benchmark problem #8. Annals of Nuclear Energy, 2020, 143, 107466. | 1.8 | 4 |
| 81 | Forced propagation method for Monte Carlo fission source convergence acceleration in the RMC. Nuclear Science and Techniques/Hewuli, 2021, 32, 1. | 3.4 | 4 |
| 82 | Hybrid windowed networks for on-the-fly Doppler broadening in RMC code. Nuclear Science and Techniques/Hewuli, 2021, 32, 1. | 3.4 | 4 |
| 83 | An improved tracking method for particle transport Monte Carlo simulations. Journal of Computational Physics, 2021, 437, 110330. | 3.8 | 4 |
| 84 | Source extrapolation scheme for Monte Carlo fission source convergence based on RMC code. Annals of Nuclear Energy, 2022, 166, 108737. | 1.8 | 4 |
| 85 | A transient multiphysics coupling method based on OpenFOAM for heat pipe cooled reactors. Science China Technological Sciences, 2022, 65, 102. | 4.0 | 4 |
| 86 | Study on integrated TRU multi-recycling in sodium cooled fast reactor CDFR. Nuclear Engineering and Design, 2010, 240, 3638-3644. | 1.7 | 3 |
| 87 | An advanced approach to calculation of neutron resonance self-shielding. Nuclear Engineering and Design, 2011, 241, 3051-3057. | 1.7 | 3 |
| 88 | Power optimization for application-specific 3D network-on-chip with multiple supply voltages. , 2013, , . | | 3 |
| 89 | Generalized Sensitivity Analysis With Continuous-Energy Monte Carlo Code RMC. , 2016, , . | | 3 |
| 90 | Prediction of Flow and Temperature Distributions in a High Flux Research Reactor Using the Porous Media Approach. Science and Technology of Nuclear Installations, 2017, 2017, 1-13. | 0.8 | 3 |

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| 91 | A hash mapping method using cell vectors in Monte Carlo code RMC. Annals of Nuclear Energy, 2021, 160, 108395. | 1.8 | 3 |
| 92 | ODR-VS method for a high packing fraction of dispersed TRISO particles. Annals of Nuclear Energy, 2022, 166, 108821. | 1.8 | 3 |
| 93 | Research on Monte Carlo Perturbation Calculation Methods Applied in Reactor Physics. , 2009, , . | | 2 |
| 94 | A Method of Optimized Utilization of Point-Wise Data Format in Monte Carlo Code. , 2010, , . | | 2 |
| 95 | Matrix Method of Characteristics Based on Modular Ray Tracing. , 2013, , . | | 2 |
| 96 | The numerical solution of space-dependent neutron kinetics equations in hexagonal-z geometry using Diagonally Implicit Runge Kutta method. Annals of Nuclear Energy, 2016, 94, 150-154. | 1.8 | 2 |
| 97 | Comparison of sensitivity calculation between mathematical and physical adjoint in the 2-D/1-D transport solver KYADJ. Annals of Nuclear Energy, 2019, 130, 357-364. | 1.8 | 2 |
| 98 | Development of the integrated parallelism strategy for large scale depletion calculation in the Monte Carlo code RMC. Annals of Nuclear Energy, 2020, 135, 106941. | 1.8 | 2 |
| 99 | Geometric sensitivity analysis of generalized response function with RMC code. Annals of Nuclear Energy, 2020, 149, 107824. | 1.8 | 2 |
| 100 | Improved generalized perturbation theory method for sensitivity analysis of generalized response function. Progress in Nuclear Energy, 2021, 134, 103643. | 2.9 | 2 |
| 101 | A New Method on Flux Mapping by Function Expansion. , 2010, , . | | 2 |
| 102 | RMC1.0: Development of Monte Carlo Code for Reactor Analysis. , 2010, , . | | 2 |
| 103 | The Development and Validation of Nuclear Cross Section Processing Code for Reactor-RXSP. , 2013, , . | | 2 |
| 104 | Nuclear Data Statistical Treatment. , 2013, , . | | 2 |
| 105 | ICONE19-43353 DEVELOPMENT AND VALIDATION OF BURNUP FUNCTION IN REACTOR MONTE CARLO RMC. The Proceedings of the International Conference on Nuclear Engineering (ICONE), 2011, 2011.19, _ICONE1943-_ICONE1943. | 0.0 | 2 |
| 106 | A new functional expansion tallies (FET) method based on cutting track-length estimation in RMC code. Nuclear Engineering and Design, 2022, 391, 111736. | 1.7 | 2 |
| 107 | Development of an improved direct kinetic simulation capability in RMC code. Annals of Nuclear Energy, 2022, 173, 109110. | 1.8 | 2 |
| 108 | Neutron Generation Based Method for Monte Carlo Three-Dimensional Reactor Time-Dependent Simulation. , 2012, , . | | 1 |

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| 109 | Leakage-aware performance-driven TSV-planning based on network flow algorithm in 3D ICs. , 2012, , . | | 1 |
| 110 | Post-floorplanning power optimization for MSV-driven application specific NoC design. , 2014, , . | | 1 |
| 111 | The least-squares method based on coupling coefficients for reactor power distribution reconstruction. Annals of Nuclear Energy, 2016, 94, 272-278. | 1.8 | 1 |
| 112 | Equivalence treatment in homogenization via Monte Carlo method. Annals of Nuclear Energy, 2016, 92, 72-80. | 1.8 | 1 |
| 113 | Analysis and solution of current spike occurred in dynamic compensation of self-powered neutron detectors. Annals of Nuclear Energy, 2017, 101, 83-88. | 1.8 | 1 |
| 114 | Newly Developed Coupling Scheme in RMC With Internal Thermal Feedback and OTF Doppler-Broadening Method. , 2017, , . | | 1 |
| 115 | Internal Coupling Between Neutronics and Thermal-Hydraulics With RMC/CTF and Validation Using VERA Benchmarks. , 2018, , . | | 1 |
| 116 | Comparative analysis of coupling schemes of Monte Carlo burnup calculation in RMC. Annals of Nuclear Energy, 2020, 137, 107024. | 1.8 | 1 |
| 117 | RMC/ANSYS MULTI-PHYSICS COUPLING SOLUTIONS FOR HEAT PIPE COOLED REACTORS ANALYSES. EPJ Web of Conferences, 2021, 247, 06007. | 0.3 | 1 |
| 118 | ON-THE-FLY INTERPOLATION OF CONTINUOUS TEMPERATURE-DEPENDENT THERMAL NEUTRON SCATTERING DATA IN RMC CODE. EPJ Web of Conferences, 2021, 247, 09012. | 0.3 | 1 |
| 119 | Temperature perturbation method using on-the-fly treatment of the cross-sections in the resolved resonance region. Annals of Nuclear Energy, 2021, 159, 108329. | 1.8 | 1 |
| 120 | ICONE23-1215 APPLICATION OF WIELANDT METHOD IN CONTINUOUS-ENERGY NUCLEAR DATA SENSITIVITY ANALYSIS WITH RMC CODE. The Proceedings of the International Conference on Nuclear Engineering (ICONE), 2015, 2015.23, _ICONE23-1-_ICONE23-1. | 0.0 | 1 |
| 121 | Single neutron tracking method for calculating the probability of survival based on the RMC code. Annals of Nuclear Energy, 2022, 165, 108763. | 1.8 | 1 |
| 122 | Research on global neighbor list method in Monte Carlo code RMC. Annals of Nuclear Energy, 2022, 167, 108861. | 1.8 | 1 |
| 123 | A new pin-resolved ultra-fine-group method based on globalâ€œlocal resonance treatment framework. Annals of Nuclear Energy, 2022, 170, 108954. | 1.8 | 1 |
| 124 | Development of MCBurn and its application in the analysis of SCWR physical characteristics. Frontiers of Energy and Power Engineering in China, 2009, 3, 348-352. | 0.4 | 0 |
| 125 | Thermal Neutron Cross Sections of Cold-Moderators. , 2010, , . | | 0 |
| 126 | Acceleration and Parallelization of Arbitrary Trajectory Based Three-Dimensional Method of Characteristics. , 2012, , . | | 0 |

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| 127 | The Modelling and Coupling Methodology of ANSYS CFX Using Porous Media for PB-AHTR. , 2013, , . | | 0 |
| 128 | A Simplified Supercritical Fast Reactor with Thorium Fuel. Science and Technology of Nuclear Installations, 2014, 2014, 1-9. | 0.8 | 0 |
| 129 | A Three-Dimensional Flux Expansion Nodal Method for Hexagonal Geometry Application. , 2016, , . | | 0 |
| 130 | Verification of Shielding Calculation Capability of RMC With H.B.Robinson-2 Pressure Vessel Benchmark. , 2018, , . | | 0 |
| 131 | One Step Method for Multigroup Adjoint Neutron Flux Through Continuous Energy Monte Carlo Calculation. , 2018, , . | | 0 |
| 132 | Parallelization and optimization of RMC for criticality computing based on the heterogeneous architecture of the Sunway TaihuLight supercomputer. Annals of Nuclear Energy, 2020, 147, 107761. | 1.8 | 0 |
| 133 | Verification of on-the-fly homogenization method based on the analysis of HTTR. Annals of Nuclear Energy, 2021, 150, 107831. | 1.8 | 0 |
| 134 | GENERALIZED SENSITIVITY ANALYSIS CAPABILITY WITH THE DIFFERENTIAL OPERATOR METHOD IN RMC CODE. EPJ Web of Conferences, 2021, 247, 15020. | 0.3 | 0 |
| 135 | Optimizing the RMC Code Using the Decay Chain Method for Large-Scale Decay Calculations. Frontiers in Energy Research, 2021, 9, . | 2.3 | 0 |
| 136 | Optimization of spatial structure designs of control rod using Monte Carlo code RMC. Frontiers in Energy, 0, , 1. | 2.3 | 0 |
| 137 | AN IMPROVED DISTINCT ELEMENT METHOD FOR HIGH PACKING FRACTION STOCHASTIC MEDIA MODELING. EPJ Web of Conferences, 2021, 247, 04026. | 0.3 | 0 |
| 138 | Research on the Control System of Thorium-Based Long-Life Core. , 2008, , . | | 0 |
| 139 | Leakage-Aware TSV-Planning with Power-Temperature-Delay Dependence in 3D ICs. IEICE Transactions on Fundamentals of Electronics, Communications and Computer Sciences, 2011, E94-A, 2490-2498. | 0.3 | 0 |
| 140 | 3D Neutron Transport PWR Full-core Calculation with RMC code. , 2014, , . | | 0 |
| 141 | Validation of a New PURC Module for Calculating Probability Table in Unresolved Resonance Region in RXSP Code. , 2014, , . | | 0 |
| 142 | 233U Evaluation Comparison Study. , 2014, , . | | 0 |
| 143 | The Super Equivalence Method in Monte Carlo Based Homogenization. , 2014, , . | | 0 |
| 144 | ICONE23-1418 DEVELOPMENT AND VALIDATION OF THE FAST DOPPLER BROADENING MODULE COUPLED WITHIN RMC CODE. The Proceedings of the International Conference on Nuclear Engineering (ICONE), 2015, 2015.23, _ICONE23-1-_ICONE23-1. | 0.0 | 0 |

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| 145 | INVERSE UNCERTAINTY QUANTIFICATION OF CTF PHYSICAL MODEL PARAMETERS USING BAYESIAN INFERENCE. The Proceedings of the International Conference on Nuclear Engineering (ICONE), 2019, 2019.27, 1435. | 0.0 | 0 |
| 146 | EXPLORATION OF TRAINING MODE FOR PHD STUDENTS OF NUCLEAR ENGINEERING. The Proceedings of the International Conference on Nuclear Engineering (ICONE), 2019, 2019.27, 1376. | 0.0 | 0 |
| 147 | On-the-fly temperature-dependent cross section treatment under extremes in RMC code. EPJ Web of Conferences, 2020, 239, 22009. | 0.3 | 0 |
| 148 | Application of homogenization techniques for inflow transport approximation on light water reactor analysis. Nuclear Science and Techniques/Hewuli, 2022, 33, 1. | 3.4 | 0 |
| 149 | Verification of CENDL-3.2 and ENDF/B-VIII.0 Evaluated Nuclear Data Library on HTR-10 Benchmark. Frontiers in Energy Research, 2022, 9, . | 2.3 | 0 |
| 150 | Polynomial interpolation cross-section parameterization method with the RMC Monte Carlo code. Annals of Nuclear Energy, 2022, 174, 109161. | 1.8 | 0 |