Christian Linsmeier

List of Publications by Year in Descending Order

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The third column is the impact factor (IF) of the journal, and the fourth column is the number of citations of the article.

60 293 5,495 37 h-index g-index citations papers 2.6 6,319 301 5.57 L-index avg, IF ext. papers ext. citations

#	Paper	IF	Citations
293	Beryllium erosion and redeposition in ITER H, He and DII discharges. <i>Nuclear Fusion</i> , 2022 , 62, 036011	3.3	1
292	Predictive 3D modelling of erosion and deposition in ITER with ERO2.0: from beryllium main wall, tungsten divertor to full-tungsten device. <i>Physica Scripta</i> , 2022 , 97, 014001	2.6	
291	Analysis of trapping sites for deuterium in WIIrII SMART alloy. <i>Vacuum</i> , 2022 , 199, 110956	3.7	O
290	Manufacturing of W/steel composites using electro-discharge sintering process. <i>Nuclear Materials and Energy</i> , 2021 , 30, 101089	2.1	1
289	Influence of neon seeding on the deuterium retention and surface modification of ITER-like forged tungsten. <i>Nuclear Fusion</i> , 2021 , 61, 016007	3.3	2
288	Tungsten fiber reinforced tungsten (Wf/W) using yarn based textile preforms. <i>Physica Scripta</i> , 2021 , 96, 124063	2.6	0
287	Manufacturing of W-steel joint using plasma sprayed graded W/steel-interlayer with current assisted diffusion bonding. <i>Fusion Engineering and Design</i> , 2021 , 172, 112896	1.7	1
286	A sensitivity analysis of numerical predictions for beryllium erosion and migration in ITER. <i>Nuclear Materials and Energy</i> , 2021 , 26, 100904	2.1	4
285	Self-passivating smart tungsten alloys for DEMO: a progress in joining and upscale for a first wall mockup. <i>Tungsten</i> , 2021 , 3, 101-115	4.6	2
284	Hydrogen isotope permeation through yttria coatings on Eurofer in the diffusion limited regime. <i>International Journal of Hydrogen Energy</i> , 2021 , 46, 13142-13149	6.7	1
283	Smart alloys as armor material for DEMO: Overview of properties and joining to structural materials. <i>Fusion Engineering and Design</i> , 2021 , 166, 112272	1.7	3
282	Characteristics of Microstructure Evolution during FAST Joining of the Tungsten Foil Laminate. <i>Metals</i> , 2021 , 11, 886	2.3	1
281	Design improvements, assembly and testing of the ICRH antenna for W7-X. Fusion Engineering and Design, 2021 , 166, 112205	1.7	1
280	Data on erosion and hydrogen fuel retention in Beryllium plasma-facing materials. <i>Nuclear Materials and Energy</i> , 2021 , 27, 100994	2.1	7
279	Progress on MATEO probe heads and observation system. Fusion Engineering and Design, 2021, 167, 11	22 <i>9</i> 7	O
278	The impact of surface morphology on the erosion of metallic surfaces [Modelling with the 3D Monte-Carlo code ERO2.0. <i>Nuclear Materials and Energy</i> , 2021 , 27, 100987	2.1	6
277	Design of tungsten fiber-reinforced tungsten composites with porous matrix. <i>Materials Science</i> & <i>amp; Engineering A: Structural Materials: Properties, Microstructure and Processing</i> , 2021 , 817, 141361	5.3	4

(2020-2021)

276	Advanced design of the ITER core CXRS shutter and integration into the diagnostic shield module of the Upper Port Plug No. 3. <i>Fusion Engineering and Design</i> , 2021 , 168, 112391	1.7	О	
275	Fusion R eactor Materials 2021 , 594-619			
274	The upgraded TOMAS device: A toroidal plasma facility for wall conditioning, plasma production, and plasma-surface interaction studies. <i>Review of Scientific Instruments</i> , 2021 , 92, 023506	1.7	3	
273	A New High-Throughput Focused MeV Ion-Beam Analysis Setup. <i>Instruments</i> , 2021 , 5, 10	1.2	2	
272	In situ study of short-term retention of deuterium in tungsten during and after plasma exposure in PSI-2. <i>Nuclear Fusion</i> , 2021 , 61, 096006	3.3	3	
271	Improving the W Coating Uniformity by a COMSOL Model-Based CVD Parameter Study for Denser Wf/W Composites. <i>Metals</i> , 2021 , 11, 1089	2.3	2	
270	Simultaneous irradiation and thermal effects on 16 MeV proton irradiated tungsten samples. <i>Physica Scripta</i> , 2021 , 96, 124014	2.6		
269	On grain growth and phase precipitation behaviors during W-Cr-Zr alloy densification using field-assisted sintering technology. <i>International Journal of Refractory Metals and Hard Materials</i> , 2021 , 98, 105552	4.1	1	
268	Influence of the applied pressure on the microstructure evolution of W-Cr-Y-Zr alloys during the FAST process. <i>Fusion Engineering and Design</i> , 2021 , 169, 112474	1.7	1	
267	Advanced Self-Passivating Alloys for an Application under Extreme Conditions. <i>Metals</i> , 2021 , 11, 1255	2.3	О	
266	Characterization of neutral particle fluxes from ICWC and ECWC plasmas in the TOMAS facility. <i>Physica Scripta</i> , 2021 , 96, 124025	2.6	1	
265	Modeling and experimental validation of a Wf/W-fabrication by chemical vapor deposition and infiltration. <i>Nuclear Materials and Energy</i> , 2021 , 28, 101048	2.1	2	
264	Derivation of an improved semi-empirical expression for the re-ionisation background in low energy ion scattering spectra. <i>IOP SciNotes</i> , 2021 , 2, 035206	1.2		
263	Short-term retention in metallic PFCs: modelling in view of mass spectrometry and LIBS. <i>Physica Scripta</i> , 2021 , 96, 124079	2.6		
262	Hydrogen permeation and retention in deuterium plasma exposed 316L ITER steel. <i>Nuclear Materials and Energy</i> , 2020 , 25, 100878	2.1		
261	Erosion and screening of tungsten during inter/intra-ELM periods in the JET-ILW divertor. <i>Nuclear Materials and Energy</i> , 2020 , 25, 100859	2.1	2	
260	A high temperature dual-mode quartz crystal microbalance technique for erosion and thermal desorption spectroscopy measurements. <i>Review of Scientific Instruments</i> , 2020 , 91, 125104	1.7	2	
259	Tungstenthromiumttrium alloys as first wall armor material: Yttrium concentration, oxygen content and transmutation elements. Fusion Engineering and Design, 2020, 158, 111667	1.7	4	

258	Tungsten nitride as tritium permeation barrier. <i>Nuclear Materials and Energy</i> , 2020 , 24, 100752	2.1	1
257	ERO2.0 modelling of the effects of surface roughness on molybdenum erosion and redeposition in the PSI-2 linear plasma device. <i>Physica Scripta</i> , 2020 , T171, 014057	2.6	14
256	Segregation and preferential sputtering of Cr in WCrY smart alloy. <i>Nuclear Materials and Energy</i> , 2020 , 22, 100736	2.1	2
255	Fiber Volume Fraction Influence on Randomly Distributed Short Fiber Tungsten Fiber-Reinforced Tungsten Composites. <i>Advanced Engineering Materials</i> , 2020 , 22, 1901242	3.5	5
254	Overview of challenges and developments in joining tungsten and steel for future fusion reactors. <i>Physica Scripta</i> , 2020 , T171, 014028	2.6	16
253	CRDS modelling of deuterium release from co-deposited beryllium layers in temperature programmed and laser induced desorption experiments. <i>Physica Scripta</i> , 2020 , T171, 014053	2.6	2
252	The use of tungsten yarns in the production for W f /W. Physica Scripta, 2020, T171, 014061	2.6	2
251	Peculiarity of highly radiating multi-impurity seeded H-mode plasmas on JET with ITER-like wall. <i>Physica Scripta</i> , 2020 , T171, 014055	2.6	3
250	Development of tungsten fiber-reinforced tungsten with a porous matrix. <i>Physica Scripta</i> , 2020 , T171, 014030	2.6	4
249	Smart Tungsten-based Alloys for a First Wall of DEMO. Fusion Engineering and Design, 2020, 159, 11174	2 1.7	8
248	First Monte-Carlo modelling of global beryllium migration in ITER using ERO2.0. <i>Contributions To Plasma Physics</i> , 2020 , 60, e201900149	1.4	9
247	On the plasma suitability of WCrY smart alloys t he effect of mixed D+Ar/He plasmas. <i>Physica Scripta</i> , 2020 , T171, 014002	2.6	4
246	Reversed-slit spectroscopy method for in situ measurement of H isotopes on plasma facing material. <i>Journal of Instrumentation</i> , 2020 , 15, C01007-C01007	1	
2.45			
245	Preliminary study of a visible, high spatial resolution spectrometer for DEMO divertor survey. Journal of Instrumentation, 2020 , 15, C01008-C01008	1	2
244		8.1	5
	Microstructural and micromechanical assessment of aged ultra-fast sintered functionally graded		
244	Microstructural and micromechanical assessment of aged ultra-fast sintered functionally graded iron/tungsten composites. Materials and Design, 2020, 191, 108652 The influence of heating rate on W-Cr-Zr alloy densification process and microstructure evolution	8.1	5

(2019-2020)

240	An in situ diagnostic method for monitoring of fuel retention on the first wall under long-pulse operation of experimental advanced superconducting tokamak. <i>Physica Scripta</i> , 2020 , T171, 014069	2.6	3
239	Investigation of laser ablation features of molybdenum bulk for picosecond laser-based techniques in fusion devices. <i>Fusion Engineering and Design</i> , 2020 , 151, 111379	1.7	11
238	Indentation testing on 3 MeV proton irradiated tungsten. Nuclear Materials and Energy, 2020, 25, 100776	ź .1	2
237	Temporally resolved LEIS measurements of Cr segregation after preferential sputtering of WCrY alloy. <i>Nuclear Instruments & Methods in Physics Research B</i> , 2020 , 479, 42-46	1.2	О
236	Micro-structured tungsten, a high heat flux pulse proof material. <i>Nuclear Materials and Energy</i> , 2020 , 25, 100789	2.1	2
235	Modeling and validation of chemical vapor deposition of tungsten for tungsten fiber reinforced tungsten composites. <i>Surface and Coatings Technology</i> , 2020 , 381, 124745	4.4	7
234	Emission of Fast Hydrogen Atoms in a Low Density Gas DischargeThe Most NaturallMirror Laboratory. <i>Atoms</i> , 2019 , 7, 81	2.1	1
233	Quantitative analysis of elemental depth on Wendelstein 7-X divertor baffle screws by picosecond laser-induced breakdown spectroscopy. <i>Spectrochimica Acta, Part B: Atomic Spectroscopy</i> , 2019 , 160, 105689	3.1	10
232	Modelling of tungsten erosion and deposition in the divertor of JET-ILW in comparison to experimental findings. <i>Nuclear Materials and Energy</i> , 2019 , 18, 239-244	2.1	14
231	Diffusion model of the impact of helium and argon impurities on deuterium retention in tungsten. Nuclear Fusion, 2019 , 59, 046004	3.3	6
230	On the nature of carbon embrittlement of tungsten fibers during powder metallurgical processes. <i>Fusion Engineering and Design</i> , 2019 , 145, 18-22	1.7	13
229	Beryllium global erosion and deposition at JET-ILW simulated with ERO2.0. <i>Nuclear Materials and Energy</i> , 2019 , 18, 331-338	2.1	24
228	Fracture behavior of random distributed short tungsten fiber-reinforced tungsten composites. Nuclear Fusion, 2019, 59, 086034	3.3	6
227	Spectroscopic studies of fuel recycling and impurity behaviors in the divertor region of Wendelstein 7-X. <i>Plasma Science and Technology</i> , 2019 , 21, 105102	1.5	2
226	Endoscopes for observation of plasma-wall interactions in the divertor of Wendelstein 7-X. <i>Fusion Engineering and Design</i> , 2019 , 146, 19-22	1.7	1
225	Improved ERO modelling of beryllium erosion at ITER upper first wall panel using JET-ILW and PISCES-B experience. <i>Nuclear Materials and Energy</i> , 2019 , 19, 510-515	2.1	10
224	Modeling of H/D isotope-exchange in crystalline beryllium. <i>Nuclear Materials and Energy</i> , 2019 , 20, 10068	32 í	1
223	Optimization of single crystal mirrors for ITER diagnostics. <i>Fusion Engineering and Design</i> , 2019 , 146, 1450-1453	1.7	5

222	On the use of rhodium mirrors for optical diagnostics in ITER. <i>Fusion Engineering and Design</i> , 2019 , 146, 2514-2518	1.7	5
221	Laser-Induced Desorption of co-deposited Deuterium in Beryllium Layers on Tungsten. <i>Nuclear Materials and Energy</i> , 2019 , 19, 503-509	2.1	6
220	Surface roughness effect on Mo physical sputtering and re-deposition in the linear plasma device PSI-2 predicted by ERO2.0. <i>Nuclear Materials and Energy</i> , 2019 , 19, 13-18	2.1	18
219	Atmospheric plasma spraying of functionally graded steel/tungsten layers for the first wall of future fusion reactors. <i>Surface and Coatings Technology</i> , 2019 , 366, 170-178	4.4	27
218	Materials development for new high heat-flux component mock-ups for DEMO. <i>Fusion Engineering and Design</i> , 2019 , 146, 1431-1436	1.7	11
217	Sublimation of advanced tungsten alloys under DEMO relevant accidental conditions. <i>Fusion Engineering and Design</i> , 2019 , 146, 1198-1202	1.7	8
216	Conceptual studies on spectroscopy and radiation diagnostic systems for plasma control on DEMO. <i>Fusion Engineering and Design</i> , 2019 , 146, 2297-2301	1.7	6
215	Comparison of the hydrogen permeation through fusion relevant steels and the influence of oxidized and rough surfaces. <i>Nuclear Materials and Energy</i> , 2019 , 19, 55-58	2.1	10
214	Ecro-structured tungsten: an advanced plasma-facing material. <i>Nuclear Materials and Energy</i> , 2019 , 19, 7-12	2.1	10
213	On the possibility of track length based Monte-Carlo algorithms for stationary drift-diffusion systems with sources and sinks. <i>Journal of Computational Physics</i> , 2019 , 377, 219-231	4.1	3
212	Insight into single-fiber push-out test of tungsten fiber-reinforced tungsten. <i>Composite Interfaces</i> , 2019 , 26, 107-126	2.3	5
211	Diffusivity of hydrogen and properties of point defects in beryllium investigated by DFT. <i>Journal of Nuclear Materials</i> , 2019 , 524, 323-329	3.3	4
210	Fuel Retention Diagnostic Setup (FREDIS) for desorption of gases from beryllium and tritium containing samples. <i>Fusion Engineering and Design</i> , 2019 , 146, 1176-1180	1.7	4
209	Argon-seeded plasma exposure and oxidation performance of tungsten-chromium-yttrium smart alloys. <i>Tungsten</i> , 2019 , 1, 159-168	4.6	7
208	Preferential sputtering induced Cr-Diffusion during plasma exposure of WCrY smart alloys. <i>Journal of Nuclear Materials</i> , 2019 , 526, 151767	3.3	3
207	Influence of plasma impurities on the fuel retention in tungsten. <i>Nuclear Fusion</i> , 2019 , 59, 086029	3.3	13
206	Erosion and deposition investigations on Wendelstein 7-X first wall components for the first operation phase in divertor configuration. <i>Fusion Engineering and Design</i> , 2019 , 146, 242-245	1.7	13
205	Ultra-fast sintered functionally graded Fe/W composites for the first wall of future fusion reactors. <i>Composites Part B: Engineering</i> , 2019 , 164, 205-214	10	33

(2018-2019)

204	An upgraded LIBS system on linear plasma device PSI-2 for in situ diagnostics of plasma-facing materials. <i>Fusion Engineering and Design</i> , 2019 , 146, 96-99	1.7	8
203	Plastic deformation of tungsten due to deuterium plasma exposure: Insights from micro-compression tests. <i>Scripta Materialia</i> , 2019 , 162, 132-135	5.6	10
202	Evaluation of the high temperature oxidation of W-Cr-Zr self-passivating alloys. <i>Corrosion Science</i> , 2019 , 147, 201-211	6.8	15
201	Design status of the ITER core CXRS diagnostic setup. Fusion Engineering and Design, 2019, 146, 228-23	11.7	3
200	Diagnostic setup for the divertor manipulator at wendelstein 7-X. <i>Nuclear Materials and Energy</i> , 2019 , 18, 77-81	2.1	5
199	Determination of tungsten sources in the JET-ILW divertor by spectroscopic imaging in the presence of a strong plasma continuum. <i>Nuclear Materials and Energy</i> , 2019 , 18, 118-124	2.1	9
198	Depth resolved analysis of hydrogen in W7-X graphite components using laser-induced ablation-quadrupole mass spectrometry (LIA-QMS). <i>Nuclear Materials and Energy</i> , 2019 , 18, 153-158	2.1	11
197	Smart first wall materials for intrinsic safety of a fusion power plant. <i>Fusion Engineering and Design</i> , 2018 , 136, 878-882	1.7	10
196	Probe manipulators for Wendelstein 7-X and their interaction with the magnetic topology. <i>Plasma Science and Technology</i> , 2018 , 20, 054002	1.5	
195	Influence of the interface strength on the mechanical properties of discontinuous tungsten fiber-reinforced tungsten composites produced by field assisted sintering technology. <i>Composites Part A: Applied Science and Manufacturing</i> , 2018 , 107, 342-353	8.4	46
194	Ablation mass features in multi-pulses femtosecond laser ablate molybdenum target. <i>Nuclear Instruments & Methods in Physics Research B</i> , 2018 , 418, 54-59	1.2	8
193	Crack bridging in as-fabricated and embrittled tungsten single fibre-reinforced tungsten composites shown by a novel in-situ high energy synchrotron tomography bending test. <i>Nuclear Materials and Energy</i> , 2018 , 15, 1-12	2.1	13
192	Depth-resolved sample composition analysis using laser-induced ablation-quadrupole mass spectrometry and laser-induced breakdown spectroscopy. <i>Spectrochimica Acta, Part B: Atomic Spectroscopy</i> , 2018 , 144, 38-45	3.1	10
191	Reaction-diffusion modeling of hydrogen transport and surface effects in application to single-crystalline Be. <i>Nuclear Instruments & Methods in Physics Research B</i> , 2018 , 430, 23-30	1.2	14
190	On Oxidation Resistance Mechanisms at 1273 K of Tungsten-Based Alloys Containing Chromium and Yttria. <i>Metals</i> , 2018 , 8, 488	2.3	13
189	Aggravated blistering and increased deuterium retention in iron-damaged tungsten after exposure to deuterium plasma with various surface temperatures. <i>Nuclear Fusion</i> , 2018 , 58, 106005	3.3	9
188	An ultraviolet-visible-near infrared overview spectroscopy for divertor plasma diagnosis on Wendelstein 7-X. <i>AIP Advances</i> , 2018 , 8, 085011	1.5	5
187	Real-time protection of the JET ITER-like wall based on near infrared imaging diagnostic systems. <i>Nuclear Fusion</i> , 2018 , 58, 106021	3.3	9

186	Temperature-dependent in-situ LEIS measurement of W surface enrichment by 250 LeV D sputtering of EUROFER. <i>Nuclear Materials and Energy</i> , 2018 , 16, 181-190	2.1	9
185	Aiming at understanding thermo-mechanical loads in the first wall of DEMO: StressEtrain evolution in a Eurofer-tungsten test component featuring a functionally graded interlayer. <i>Fusion Engineering and Design</i> , 2018 , 135, 141-153	1.7	18
184	Laser induced ablation spectroscopy for in situ characterization of the first wall on EAST tokamak. <i>Fusion Engineering and Design</i> , 2018 , 135, 95-101	1.7	14
183	WCrY smart alloys as advanced plasma-facing materials Exposure to steady-state pure deuterium plasmas in PSI-2. <i>Nuclear Materials and Energy</i> , 2018 , 15, 220-225	2.1	15
182	Modelling of plasma-wall interaction and impurity transport in fusion devices and prompt deposition of tungsten as application. <i>Plasma Physics and Controlled Fusion</i> , 2018 , 60, 014041	2	21
181	Identification of BeO and BeOxDy in melted zones of the JET Be limiter tiles: Raman study using comparison with laboratory samples. <i>Nuclear Materials and Energy</i> , 2018 , 17, 295-301	2.1	11
180	Impact of Kr and Ar seeding on D retention in ferritic-martensitic steels after high-fluence plasma exposure. <i>Nuclear Materials and Energy</i> , 2018 , 17, 307-313	2.1	1
179	Influence of the grain structure of yttria thin films on the hydrogen isotope permeation. <i>International Journal of Hydrogen Energy</i> , 2018 , 43, 22976-22985	6.7	8
178	Plasma exposures of a high-conductivity graphitic foam for plasma facing components. <i>Nuclear Materials and Energy</i> , 2018 , 17, 123-128	2.1	4
177	Hydrogen embrittlement of tungsten induced by deuterium plasma: Insights from nanoindentation tests. <i>Journal of Materials Research</i> , 2018 , 33, 3530-3536	2.5	19
176	Improved pseudo-ductile behavior of powder metallurgical tungsten short fiber-reinforced tungsten (Wf/W). <i>Nuclear Materials and Energy</i> , 2018 , 15, 214-219	2.1	23
175	Oxidation resistance of bulk plasma-facing tungsten alloys. <i>Nuclear Materials and Energy</i> , 2018 , 15, 226-	·2:3: <u>1</u> 1	20
174	Modelling of deposition and erosion of injected WF6 and MoF6 in TEXTOR. <i>Nuclear Materials and Energy</i> , 2017 , 12, 564-568	2.1	2
173	Surface modification of He pre-exposed tungsten samples by He plasma impact in the divertor manipulator of ASDEX Upgrade. <i>Nuclear Materials and Energy</i> , 2017 , 12, 575-581	2.1	12
172	Design and development of a LIBS system on linear plasma device PSI-2 for in situ real-time diagnostics of plasma-facing materials. <i>Nuclear Materials and Energy</i> , 2017 , 12, 1224-1230	2.1	7
171	Smart alloys for a future fusion power plant: First studies under stationary plasma load and in accidental conditions. <i>Nuclear Materials and Energy</i> , 2017 , 12, 1363-1367	2.1	17
170	Hydrogen saturation and permeation barrier performance of yttrium oxide coatings. <i>Fusion Engineering and Design</i> , 2017 , 124, 1140-1143	1.7	20
169	A multi-purpose manipulator system for W7-X as user facility for plasma edge investigation. <i>Fusion Engineering and Design</i> , 2017 , 123, 960-964	1.7	37

(2017-2017)

168	multi-wavelengths Raman spectroscopy on implanted Be, and co-deposited Be. <i>Nuclear Fusion</i> , 2017 , 57, 076035	3.3	5	
167	Advanced smart tungsten alloys for a future fusion power plant. <i>Plasma Physics and Controlled Fusion</i> , 2017 , 59, 064003	2	21	
166	Smart tungsten alloys as a material for the first wall of a future fusion power plant. <i>Nuclear Fusion</i> , 2017 , 57, 066020	3.3	32	
165	Development of advanced high heat flux and plasma-facing materials. <i>Nuclear Fusion</i> , 2017 , 57, 092007	3.3	137	
164	Major results from the first plasma campaign of the Wendelstein 7-X stellarator. <i>Nuclear Fusion</i> , 2017 , 57, 102020	3.3	88	
163	Design of an ICRF system for plasmaWall interactions and RF plasma production studies on TOMAS. Fusion Engineering and Design, 2017, 123, 317-320	1.7	3	
162	Improving accuracy of Penning gauge spectroscopy for the determination of hydrogen isotope H/D ratios. <i>Fusion Engineering and Design</i> , 2017 , 123, 906-910	1.7	5	
161	The effect of the isotope on the H-mode density limit. <i>Nuclear Fusion</i> , 2017 , 57, 086007	3.3	8	
160	First direct comparative test of single crystal rhodium and molybdenum mirrors for ITER diagnostics. <i>Fusion Engineering and Design</i> , 2017 , 123, 674-677	1.7	13	
159	The influence of annealing on yttrium oxide thin film deposited by reactive magnetron sputtering: Process and microstructure. <i>Nuclear Materials and Energy</i> , 2017 , 10, 1-8	2.1	26	
158	Tensile deformation behavior of tungsten fibre-reinforced tungsten composite specimens in as-fabricated state. <i>Fusion Engineering and Design</i> , 2017 , 124, 396-400	1.7	32	
157	In-situ mass-spectrometer of magnetized plasmas. <i>Nuclear Materials and Energy</i> , 2017 , 12, 1243-1247	2.1	6	
156	Comparative H-mode density limit studies in JET and AUG. <i>Nuclear Materials and Energy</i> , 2017 , 12, 100-	110	7	
155	Advanced materials for a damage resilient divertor concept for DEMO: Powder-metallurgical tungsten-fibre reinforced tungsten. <i>Fusion Engineering and Design</i> , 2017 , 124, 964-968	1.7	32	
154	The microstructure of tungsten exposed to D plasma with different impurities. <i>Nuclear Materials and Energy</i> , 2017 , 12, 302-306	2.1	14	
153	Plasma-wall interaction of advanced materials. <i>Nuclear Materials and Energy</i> , 2017 , 12, 307-312	2.1	13	
152	Dynamic outgassing of deuterium, helium and nitrogen from plasma-facing materials under DEMO relevant conditions. <i>Nuclear Fusion</i> , 2017 , 57, 016020	3.3	12	
151	Tensile behaviour of drawn tungsten wire used in tungsten fibre-reinforced tungsten composites. <i>Physica Scripta</i> , 2017 , T170, 014032	2.6	13	

150	In situ investigation of helium fuzz growth on tungsten in relation to ion flux, fluence, surface temperature and ion energy using infrared imaging in PSI-2. <i>Physica Scripta</i> , 2017 , T170, 014017	2.6	7
149	Laser-induced breakdown spectroscopy for Wendelstein 7-X stellarator limiter tile analysis. <i>Physica Scripta</i> , 2017 , T170, 014004	2.6	13
148	Development and characterization of powder metallurgically produced discontinuous tungsten fiber reinforced tungsten composites. <i>Physica Scripta</i> , 2017 , T170, 014005	2.6	15
147	Deuterium retention in RAFM steels after high fluence plasma exposure. <i>Nuclear Materials and Energy</i> , 2017 , 12, 648-654	2.1	12
146	Material testing facilities and programs for plasma-facing component testing. <i>Nuclear Fusion</i> , 2017 , 57, 092012	3.3	41
145	Development and analyses of self-passivating tungsten alloys for DEMO accidental conditions. <i>Fusion Engineering and Design</i> , 2017 , 124, 183-186	1.7	25
144	Response of the imaging cameras to hard radiation during JET operation. <i>Fusion Engineering and Design</i> , 2017 , 123, 669-673	1.7	8
143	ERO modeling of beryllium erosion by helium plasma in experiments at PISCES-B. <i>Nuclear Materials and Energy</i> , 2017 , 12, 1157-1162	2.1	6
142	Theoretical investigation on the point defect formation energies in beryllium and comparison with experiments. <i>Nuclear Materials and Energy</i> , 2017 , 12, 453-457	2.1	10
141	Surface modifications and deuterium retention in polycrystalline and single crystal tungsten as a function of particle flux and temperature. <i>Journal of Nuclear Materials</i> , 2017 , 495, 211-219	3.3	14
140	Experimental data on low energy electron impact ionisation of W. <i>Physica Scripta</i> , 2017 , T170, 014075	2.6	
139	Development of laser-based technology for the routine first wall diagnostic on the tokamak EAST: LIBS and LIAS. <i>Physica Scripta</i> , 2017 , T170, 014046	2.6	8
138	The near infrared imaging system for the real-time protection of the JET ITER-like wall. <i>Physica Scripta</i> , 2017 , T170, 014027	2.6	7
137	New oxidation-resistant tungsten alloys for use in the nuclear fusion reactors. <i>Physica Scripta</i> , 2017 , T170, 014012	2.6	28
136	Diagnostic set-up and modelling for investigation of synergy between 3D edge physics and plasma-wall interactions on Wendelstein 7-X. <i>Nuclear Fusion</i> , 2017 , 57, 066049	3.3	14
135	First ERO2.0 modeling of Be erosion and non-local transport in JET ITER-like wall. <i>Physica Scripta</i> , 2017 , T170, 014018	2.6	16
134	The microstructure of reduced activation ferritic/martensitic (RAFM) steels exposed to D plasma with different seeding impurities. <i>Physica Scripta</i> , 2017 , T170, 014036	2.6	10
133	Preparation of erosion and deposition investigations on plasma facing components in Wendelstein 7-X. <i>Physica Scripta</i> , 2017 , T170, 014010	2.6	9

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132	Plasmalvall interaction studies within the EUROfusion consortium: progress on plasma-facing components development and qualification. <i>Nuclear Fusion</i> , 2017 , 57, 116041	3.3	50
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