Christian Linsmeier

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60 293 5,495 37 h-index g-index citations papers 2.6 6,319 301 5.57 L-index avg, IF ext. papers ext. citations

#	Paper	IF	Citations
293	Recent progress in research on tungsten materials for nuclear fusion applications in Europe. Journal of Nuclear Materials, 2013 , 432, 482-500	3.3	494
292	Developing structural, high-heat flux and plasma facing materials for a near-term DEMO fusion power plant: The EU assessment. <i>Journal of Nuclear Materials</i> , 2014 , 455, 277-291	3.3	172
291	Review on the EFDA programme on tungsten materials technology and science. <i>Journal of Nuclear Materials</i> , 2011 , 417, 463-467	3.3	139
290	Development of advanced high heat flux and plasma-facing materials. <i>Nuclear Fusion</i> , 2017 , 57, 092007	3.3	137
289	Carbon films and carbide formation on tungsten. Surface Science, 2000, 454-456, 78-82	1.8	108
288	Materials R&D for a timely DEMO: Key findings and recommendations of the EU Roadmap Materials Assessment Group. <i>Fusion Engineering and Design</i> , 2014 , 89, 1586-1594	1.7	99
287	In situ synchrotron tomography estimation of toughening effect by semi-ductile fibre reinforcement in a tungsten-fibre-reinforced tungsten composite system. <i>Acta Materialia</i> , 2013 , 61, 706	5 8-4 07	1 ⁹⁰
286	Major results from the first plasma campaign of the Wendelstein 7-X stellarator. <i>Nuclear Fusion</i> , 2017 , 57, 102020	3.3	88
285	Strong binding of bioactive BMP-2 to nanocrystalline diamond by physisorption. <i>Biomaterials</i> , 2006 , 27, 4547-56	15.6	88
284	Molybdenum oxide based partial oxidation catalyst: 1. Thermally induced oxygen deficiency, elemental and structural heterogeneity and the relation to catalytic performance. <i>Journal of Molecular Catalysis A</i> , 2000 , 162, 463-492		82
283	Beryllium migration in JET ITER-like wall plasmas. <i>Nuclear Fusion</i> , 2015 , 55, 063021	3.3	70
282	Formation of endothermic carbides on iron and nickel. <i>Physica Status Solidi A</i> , 2004 , 201, 881-887		69
281	Deuterium retention in tungsten exposed to low-energy, high-flux clean and carbon-seeded deuterium plasmas. <i>Journal of Nuclear Materials</i> , 2008 , 375, 192-201	3.3	68
280	Materials for DEMO and reactor applications B oundary conditions and new concepts. <i>Physica Scripta</i> , 2016 , T167, 014002	2.6	68
279	Retention mechanisms and binding states of deuterium implanted into beryllium. <i>New Journal of Physics</i> , 2009 , 11, 043023	2.9	64
278	A brief summary of the progress on the EFDA tungsten materials program. <i>Journal of Nuclear Materials</i> , 2013 , 442, S173-S180	3.3	63
277	Development of tungsten fibre-reinforced tungsten composites towards their use in DEMOBotassium doped tungsten wire. <i>Physica Scripta</i> , 2016 , T167, 014006	2.6	60

(2006-2013)

276	First nitrogen-seeding experiments in JET with the ITER-like Wall. <i>Journal of Nuclear Materials</i> , 2013 , 438, S258-S261	3.3	57
275	Binary beryllium E ungsten mixed materials. <i>Journal of Nuclear Materials</i> , 2007 , 363-365, 1129-1137	3.3	57
274	Advanced tungsten materials for plasma-facing components of DEMO and fusion power plants. <i>Fusion Engineering and Design</i> , 2016 , 109-111, 1046-1052	1.7	57
273	Interaction of nitrogen plasmas with tungsten. <i>Nuclear Fusion</i> , 2010 , 50, 025006	3.3	54
272	Enhanced toughness and stable crack propagation in a novel tungsten fibre-reinforced tungsten composite produced by chemical vapour infiltration. <i>Physica Scripta</i> , 2014 , T159, 014031	2.6	51
271	Composition and hydrogen isotope retention analysis of co-deposited C/Be layers. <i>Journal of Nuclear Materials</i> , 2005 , 337-339, 590-594	3.3	50
270	PlasmaWall interaction studies within the EUROfusion consortium: progress on plasma-facing components development and qualification. <i>Nuclear Fusion</i> , 2017 , 57, 116041	3.3	50
269	Reactions of oxygen atoms and molecules with Au, Be, and W surfaces. <i>Surface Science</i> , 2000 , 454-456, 305-309	1.8	49
268	Influence of the interface strength on the mechanical properties of discontinuous tungsten fiber-reinforced tungsten composites produced by field assisted sintering technology. <i>Composites Part A: Applied Science and Manufacturing</i> , 2018 , 107, 342-353	8.4	46
267	Quantum Modeling of Hydrogen Retention in Beryllium Bulk and Vacancies. <i>Journal of Physical Chemistry C</i> , 2010 , 114, 3588-3598	3.8	46
266	Development of yttrium-containing self-passivating tungsten alloys for future fusion power plants. <i>Nuclear Materials and Energy</i> , 2016 , 9, 394-398	2.1	45
265	Oxidation behaviour of silicon-free tungsten alloys for use as the first wall material. <i>Physica Scripta</i> , 2011 , T145, 014019	2.6	44
264	Chemically deposited tungsten fibre-reinforced tungsten I The way to a mock-up for divertor applications. <i>Nuclear Materials and Energy</i> , 2016 , 9, 75-83	2.1	43
263	Can aluminium or magnesium be a surrogate for beryllium: A critical investigation of their chemistry. <i>Fusion Engineering and Design</i> , 2013 , 88, 1718-1721	1.7	43
262	Material testing facilities and programs for plasma-facing component testing. <i>Nuclear Fusion</i> , 2017 , 57, 092012	3.3	41
261	Strong metalBupport interactions on rhodium model catalysts. <i>Applied Catalysis A: General</i> , 2011 , 391, 175-186	5.1	41
260	Carbon reaction and diffusion on Ni(111), Ni(100), and Fe(110): Kinetic parameters from x-ray photoelectron spectroscopy and density functional theory analysis. <i>Journal of Chemical Physics</i> , 2008 , 129, 084704	3.9	41
259	Surface alloying of thin beryllium films on tungsten. <i>New Journal of Physics</i> , 2006 , 8, 181-181	2.9	38

258	A multi-purpose manipulator system for W7-X as user facility for plasma edge investigation. <i>Fusion Engineering and Design</i> , 2017 , 123, 960-964	1.7	37
257	Thermal stability of titanium nitride diffusion barrier films for advanced silver interconnects. <i>Microelectronic Engineering</i> , 2004 , 76, 76-81	2.5	37
256	Silver metal organic chemical vapor deposition for advanced silver metallization. <i>Microelectronic Engineering</i> , 2005 , 82, 296-300	2.5	37
255	Implantation and erosion of nitrogen in tungsten. New Journal of Physics, 2014, 16, 093018	2.9	36
254	Overview of the JET results. <i>Nuclear Fusion</i> , 2015 , 55, 104001	3.3	34
253	Investigation of W components exposed to high thermal and high H/He fluxes. <i>Journal of Nuclear Materials</i> , 2011 , 417, 495-498	3.3	34
252	Ultra-fast sintered functionally graded Fe/W composites for the first wall of future fusion reactors. <i>Composites Part B: Engineering</i> , 2019 , 164, 205-214	10	33
251	Smart tungsten alloys as a material for the first wall of a future fusion power plant. <i>Nuclear Fusion</i> , 2017 , 57, 066020	3.3	32
250	Tensile deformation behavior of tungsten fibre-reinforced tungsten composite specimens in as-fabricated state. <i>Fusion Engineering and Design</i> , 2017 , 124, 396-400	1.7	32
249	Advanced materials for a damage resilient divertor concept for DEMO: Powder-metallurgical tungsten-fibre reinforced tungsten. <i>Fusion Engineering and Design</i> , 2017 , 124, 964-968	1.7	32
248	Mixed material formation and erosion. Journal of Nuclear Materials, 2001, 290-293, 25-32	3.3	32
247	Interaction of beryllium containing plasma with ITER materials. <i>Physica Scripta</i> , 2007 , T128, 115-120	2.6	31
246	Influence of tungsten microstructure and ion flux on deuterium plasma-induced surface modifications and deuterium retention. <i>Journal of Nuclear Materials</i> , 2015 , 463, 320-324	3.3	29
245	Beryllium deposition on International Thermonuclear Experimental Reactor first mirrors: Layer morphology and influence on mirror reflectivity. <i>Journal of Applied Physics</i> , 2007 , 102, 083302	2.5	29
244	Influence of oxygen on the carbide formation on tungsten. <i>Journal of Nuclear Materials</i> , 2001 , 290-293, 121-125	3.3	29
243	New oxidation-resistant tungsten alloys for use in the nuclear fusion reactors. <i>Physica Scripta</i> , 2017 , T170, 014012	2.6	28
242	Atmospheric plasma spraying of functionally graded steel/tungsten layers for the first wall of future fusion reactors. <i>Surface and Coatings Technology</i> , 2019 , 366, 170-178	4.4	27
241	Influence of plasma impurities on the deuterium retention in tungsten exposed in the linear plasma generator PSI-2. <i>Journal of Nuclear Materials</i> , 2015 , 463, 1021-1024	3.3	27

(2004-2016)

240	Properties of drawn W wire used as high performance fibre in tungsten fibre-reinforced tungsten composite. <i>IOP Conference Series: Materials Science and Engineering</i> , 2016 , 139, 012043	0.4	27	
239	The influence of annealing on yttrium oxide thin film deposited by reactive magnetron sputtering: Process and microstructure. <i>Nuclear Materials and Energy</i> , 2017 , 10, 1-8	2.1	26	
238	PlasmaWall interactions with nitrogen seeding in all-metal fusion devices: Formation of nitrides and ammonia. <i>Fusion Engineering and Design</i> , 2015 , 98-99, 1371-1374	1.7	26	
237	Development and analyses of self-passivating tungsten alloys for DEMO accidental conditions. <i>Fusion Engineering and Design</i> , 2017 , 124, 183-186	1.7	25	
236	Beryllium global erosion and deposition at JET-ILW simulated with ERO2.0. <i>Nuclear Materials and Energy</i> , 2019 , 18, 331-338	2.1	24	
235	Powder Metallurgical Tungsten Fiber-Reinforced Tungsten. <i>Materials Science Forum</i> , 2015 , 825-826, 125	5-1.33	24	
234	Calculation of cracking under pulsed heat loads in tungsten manufactured according to ITER specifications. <i>Journal of Nuclear Materials</i> , 2015 , 467, 165-171	3.3	23	
233	Behavior of tungsten fiber-reinforced tungsten based on single fiber push-out study. <i>Nuclear Materials and Energy</i> , 2016 , 9, 416-421	2.1	23	
232	Surface modification of molten W exposed to high heat flux helium neutral beams. <i>Journal of Nuclear Materials</i> , 2013 , 437, 297-302	3.3	23	
231	Deuterium release from implanted beryllium and beryllium oxide. <i>Journal of Nuclear Materials</i> , 2011 , 415, S724-S727	3.3	23	
230	Properties of nitrogen-implanted beryllium and its interaction with energetic deuterium. <i>Nuclear Fusion</i> , 2010 , 50, 125001	3.3	23	
229	Formation of a surface alloy in the beryllium E ungsten system. <i>Journal of Nuclear Materials</i> , 2005 , 337-339, 951-955	3.3	23	
228	Improved pseudo-ductile behavior of powder metallurgical tungsten short fiber-reinforced tungsten (Wf/W). <i>Nuclear Materials and Energy</i> , 2018 , 15, 214-219	2.1	23	
227	Cold atmospheric plasma IA new technology for spacecraft component decontamination. <i>Planetary and Space Science</i> , 2014 , 90, 60-71	2	22	
226	Ion beam-induced carbide formation at the the titanium darbon interface. <i>Nuclear Instruments & Methods in Physics Research B</i> , 2001 , 182, 218-226	1.2	22	
225	Advanced smart tungsten alloys for a future fusion power plant. <i>Plasma Physics and Controlled Fusion</i> , 2017 , 59, 064003	2	21	
224	Review of the high heat flux testing as an integrated part of W7-X divertor development. <i>Fusion Engineering and Design</i> , 2009 , 84, 848-852	1.7	21	
223	Metalorganic chemical vapor deposition of silver thin films for future interconnects by direct liquid injection system. <i>Materials Science in Semiconductor Processing</i> , 2004 , 7, 331-335	4.3	21	

222	Surface reactions on beryllium after carbon vapour deposition and thermal treatment. <i>Journal of Nuclear Materials</i> , 2001 , 290-293, 76-79	3.3	21
221	Modelling of plasma-wall interaction and impurity transport in fusion devices and prompt deposition of tungsten as application. <i>Plasma Physics and Controlled Fusion</i> , 2018 , 60, 014041	2	21
220	Hydrogen saturation and permeation barrier performance of yttrium oxide coatings. <i>Fusion Engineering and Design</i> , 2017 , 124, 1140-1143	1.7	20
219	Investigation of European tungsten materials exposed to high heat flux H/He neutral beams. Journal of Nuclear Materials, 2013 , 442, S256-S260	3.3	20
218	Retention and release mechanisms of deuterium implanted into beryllium. <i>Nuclear Instruments & Methods in Physics Research B</i> , 2011 , 269, 1266-1270	1.2	20
217	Temperature programmed desorption of 1 keV deuterium implanted into clean beryllium. <i>Physica Scripta</i> , 2007 , T128, 111-114	2.6	20
216	Oxidation resistance of bulk plasma-facing tungsten alloys. <i>Nuclear Materials and Energy</i> , 2018 , 15, 226	-231	20
215	Characterization of Electron Beam Evaporated Carbon Films and Compound Formation on Titanium and Silicon. <i>Physica Scripta</i> , 2001 , T91, 134	2.6	19
214	Hydrogen embrittlement of tungsten induced by deuterium plasma: Insights from nanoindentation tests. <i>Journal of Materials Research</i> , 2018 , 33, 3530-3536	2.5	19
213	Surface roughness effect on Mo physical sputtering and re-deposition in the linear plasma device PSI-2 predicted by ERO2.0. <i>Nuclear Materials and Energy</i> , 2019 , 19, 13-18	2.1	18
212	Aiming at understanding thermo-mechanical loads in the first wall of DEMO: StressEtrain evolution in a Eurofer-tungsten test component featuring a functionally graded interlayer. <i>Fusion Engineering and Design</i> , 2018 , 135, 141-153	1.7	18
211	Smart alloys for a future fusion power plant: First studies under stationary plasma load and in accidental conditions. <i>Nuclear Materials and Energy</i> , 2017 , 12, 1363-1367	2.1	17
210	Ion implanted deuterium retention and release from clean and oxidized beryllium. <i>Journal of Nuclear Materials</i> , 2009 , 390-391, 568-571	3.3	17
209	Tungsten sputtering and accumulation of implanted carbon and deuterium by simultaneous bombardment with D and C ions. <i>Journal of Nuclear Materials</i> , 2007 , 363-365, 1184-1189	3.3	17
208	Ion scattering and Auger electron spectroscopy analysis of alumina-supported rhodium model catalysts. <i>Surface Science</i> , 1992 , 275, 101-113	1.8	17
207	Simulation of neutron irradiation damage in tungsten using higher energy protons. <i>Nuclear Materials and Energy</i> , 2016 , 9, 29-35	2.1	17
206	Combined impact of transient heat loads and steady-state plasma exposure on tungsten. <i>Fusion Engineering and Design</i> , 2015 , 98-99, 1328-1332	1.7	16
205	Overview of challenges and developments in joining tungsten and steel for future fusion reactors. <i>Physica Scripta</i> , 2020 , T171, 014028	2.6	16

(2017-2016)

204	Modelling of Impurity Transport and PlasmalWall Interaction in Fusion Devices with the ERO Code: Basics of the Code and Examples of Application. <i>Contributions To Plasma Physics</i> , 2016 , 56, 622-627	1.4	16	
203	First ERO2.0 modeling of Be erosion and non-local transport in JET ITER-like wall. <i>Physica Scripta</i> , 2017 , T170, 014018	2.6	16	
202	Surface chemistry of first wall materials F rom fundamental data to modeling. <i>Journal of Nuclear Materials</i> , 2011 , 415, S212-S218	3.3	16	
201	Quantum study of tungsten interaction with beryllium (0001). <i>Journal of Physics: Conference Series</i> , 2008 , 117, 012002	0.3	16	
200	ARTOSS ? A New Surface Science Experiment to Study the Hydrogen Inventory in Multi-Component Materials. <i>Physica Scripta</i> , 2001 , T94, 28	2.6	16	
199	WCrY smart alloys as advanced plasma-facing materials Exposure to steady-state pure deuterium plasmas in PSI-2. <i>Nuclear Materials and Energy</i> , 2018 , 15, 220-225	2.1	15	
198	Absorption and diffusion of beryllium in graphite, beryllium carbide formation investigated by density functional theory. <i>Journal of Applied Physics</i> , 2013 , 113, 213514	2.5	15	
197	Development and characterization of powder metallurgically produced discontinuous tungsten fiber reinforced tungsten composites. <i>Physica Scripta</i> , 2017 , T170, 014005	2.6	15	
196	Quantum modeling (DFT) and experimental investigation of beryllium-tungsten alloy formation. <i>Journal of Physics Condensed Matter</i> , 2009 , 21, 355011	1.8	15	
195	Structural investigation of the Bell intermetallic system. <i>Physica Scripta</i> , 2007 , T128, 133-136	2.6	15	
194	Ion scattering analysis of alumina supported model catalysts. <i>Nuclear Instruments & Methods in Physics Research B</i> , 1992 , 64, 596-602	1.2	15	
193	Evaluation of the high temperature oxidation of W-Cr-Zr self-passivating alloys. <i>Corrosion Science</i> , 2019 , 147, 201-211	6.8	15	
192	The microstructure of tungsten exposed to D plasma with different impurities. <i>Nuclear Materials and Energy</i> , 2017 , 12, 302-306	2.1	14	
191	Modelling of tungsten erosion and deposition in the divertor of JET-ILW in comparison to experimental findings. <i>Nuclear Materials and Energy</i> , 2019 , 18, 239-244	2.1	14	
190	ERO2.0 modelling of the effects of surface roughness on molybdenum erosion and redeposition in the PSI-2 linear plasma device. <i>Physica Scripta</i> , 2020 , T171, 014057	2.6	14	
189	Reaction-diffusion modeling of hydrogen transport and surface effects in application to single-crystalline Be. <i>Nuclear Instruments & Methods in Physics Research B</i> , 2018 , 430, 23-30	1.2	14	
188	Laser induced ablation spectroscopy for in situ characterization of the first wall on EAST tokamak. <i>Fusion Engineering and Design</i> , 2018 , 135, 95-101	1.7	14	
187	Surface modifications and deuterium retention in polycrystalline and single crystal tungsten as a function of particle flux and temperature. <i>Journal of Nuclear Materials</i> , 2017 , 495, 211-219	3.3	14	

186	Diagnostic set-up and modelling for investigation of synergy between 3D edge physics and plasma-wall interactions on Wendelstein 7-X. <i>Nuclear Fusion</i> , 2017 , 57, 066049	3.3	14
185	Combined ion and electron spectroscopy study of the surface reactions of beryllium with carbon. <i>Nuclear Instruments & Methods in Physics Research B</i> , 2000 , 161-163, 411-414	1.2	14
184	Auger electron spectroscopy. Vacuum, 1994, 45, 673-690	3.7	14
183	Nitrogen retention mechanisms in tokamaks with beryllium and tungsten plasma-facing surfaces. <i>Physica Scripta</i> , 2016 , T167, 014077	2.6	14
182	First direct comparative test of single crystal rhodium and molybdenum mirrors for ITER diagnostics. <i>Fusion Engineering and Design</i> , 2017 , 123, 674-677	1.7	13
181	Plasma-wall interaction of advanced materials. <i>Nuclear Materials and Energy</i> , 2017 , 12, 307-312	2.1	13
180	Tensile behaviour of drawn tungsten wire used in tungsten fibre-reinforced tungsten composites. <i>Physica Scripta</i> , 2017 , T170, 014032	2.6	13
179	On the nature of carbon embrittlement of tungsten fibers during powder metallurgical processes. <i>Fusion Engineering and Design</i> , 2019 , 145, 18-22	1.7	13
178	Recent ASDEX Upgrade research in support of ITER and DEMO. <i>Nuclear Fusion</i> , 2015 , 55, 104010	3.3	13
177	Hydrogen retention in beryllium: concentration effect and nanocrystalline growth. <i>Journal of Physics Condensed Matter</i> , 2015 , 27, 475401	1.8	13
176	Crack bridging in as-fabricated and embrittled tungsten single fibre-reinforced tungsten composites shown by a novel in-situ high energy synchrotron tomography bending test. <i>Nuclear Materials and Energy</i> , 2018 , 15, 1-12	2.1	13
175	On Oxidation Resistance Mechanisms at 1273 K of Tungsten-Based Alloys Containing Chromium and Yttria. <i>Metals</i> , 2018 , 8, 488	2.3	13
174	Influence of plasma impurities on the fuel retention in tungsten. <i>Nuclear Fusion</i> , 2019 , 59, 086029	3.3	13
173	Laser-induced breakdown spectroscopy for Wendelstein 7-X stellarator limiter tile analysis. <i>Physica Scripta</i> , 2017 , T170, 014004	2.6	13
172	Structure-dependent deuterium release from ion implanted beryllium: Comparison between Be(11. <i>Nuclear Instruments & Methods in Physics Research B</i> , 2009 , 267, 718-722	1.2	13
171	Measurement of beryllium depth profiles in carbon. <i>Nuclear Instruments & Methods in Physics Research B</i> , 2004 , 219-220, 947-952	1.2	13
170	Erosion and deposition investigations on Wendelstein 7-X first wall components for the first operation phase in divertor configuration. <i>Fusion Engineering and Design</i> , 2019 , 146, 242-245	1.7	13
169	Surface modification of He pre-exposed tungsten samples by He plasma impact in the divertor manipulator of ASDEX Upgrade. <i>Nuclear Materials and Energy</i> , 2017 , 12, 575-581	2.1	12

Dynamic outgassing of deuterium, helium and nitrogen from plasma-facing materials under DEMO relevant conditions. <i>Nuclear Fusion</i> , 2017 , 57, 016020	3.3	12	
Erosion behavior of actively cooled tungsten under H/He high heat flux load. <i>Journal of Nuclear Materials</i> , 2013 , 438, S921-S924	3.3	12	
Oxidation of beryllium and exposure of beryllium oxide to deuterium plasmas in PISCES B. <i>Journal of Nuclear Materials</i> , 2013 , 438, S1044-S1047	3.3	12	
Deuterium retention in RAFM steels after high fluence plasma exposure. <i>Nuclear Materials and Energy</i> , 2017 , 12, 648-654	2.1	12	
Time resolved imaging of laser induced ablation spectroscopy (LIAS) in TEXTOR and comparison with modeling. <i>Physica Scripta</i> , 2016 , T167, 014034	2.6	12	
Materials development for new high heat-flux component mock-ups for DEMO. Fusion Engineering and Design, 2019 , 146, 1431-1436	1.7	11	
Deuterium trapping and release in Be(0001), Be(1100) and polycrystalline beryllium. <i>Journal of Nuclear Materials</i> , 2013 , 438, S1072-S1075	3.3	11	
Adsorption of beryllium atoms and clusters both on graphene and in a bilayer of graphite investigated by DFT. <i>Journal of Physics Condensed Matter</i> , 2013 , 25, 015002	1.8	11	
Towards a detailed understanding of the mechanisms of hydrogen retention in beryllium. <i>Physica Scripta</i> , 2009 , T138, 014036	2.6	11	
Thermally induced reaction and diffusion of carbon films on Ni(1 1 1) and Ni(1 0 0). <i>Surface Science</i> , 2008 , 602, 3623-3631	1.8	11	
Deuterium bombardment of carbon and carbon layers on titanium. <i>Journal of Nuclear Materials</i> , 2003 , 313-316, 56-61	3.3	11	
Formation of mixed layers and compounds on beryllium due to C+ and CO+ bombardment. <i>Journal of Nuclear Materials</i> , 2001 , 290-293, 71-75	3.3	11	
Investigation of laser ablation features of molybdenum bulk for picosecond laser-based techniques in fusion devices. <i>Fusion Engineering and Design</i> , 2020 , 151, 111379	1.7	11	
Depth resolved analysis of hydrogen in W7-X graphite components using laser-induced ablation-quadrupole mass spectrometry (LIA-QMS). <i>Nuclear Materials and Energy</i> , 2019 , 18, 153-158	2.1	11	
Identification of BeO and BeOxDy in melted zones of the JET Be limiter tiles: Raman study using comparison with laboratory samples. <i>Nuclear Materials and Energy</i> , 2018 , 17, 295-301	2.1	11	
Quantitative analysis of elemental depth on Wendelstein 7-X divertor baffle screws by picosecond laser-induced breakdown spectroscopy. <i>Spectrochimica Acta, Part B: Atomic Spectroscopy</i> , 2019 , 160, 105689	3.1	10	
Improved ERO modelling of beryllium erosion at ITER upper first wall panel using JET-ILW and PISCES-B experience. <i>Nuclear Materials and Energy</i> , 2019 , 19, 510-515	2.1	10	
Comparison of the hydrogen permeation through fusion relevant steels and the influence of oxidized and rough surfaces. <i>Nuclear Materials and Energy</i> , 2019 , 19, 55-58	2.1	10	
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64	Conceptual study of ferromagnetic pebbles for heat exhaust in fusion reactors with short power decay length. <i>Nuclear Materials and Energy</i> , 2015 , 2, 12-19	2.1	3
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59	An in situ diagnostic method for monitoring of fuel retention on the first wall under long-pulse operation of experimental advanced superconducting tokamak. <i>Physica Scripta</i> , 2020 , T171, 014069	2.6	3
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45	The approach to diamond growth on levitating seed particles. <i>Applied Surface Science</i> , 2007 , 254, 177-18	8 6 .7	2
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38	A New High-Throughput Focused MeV Ion-Beam Analysis Setup. <i>Instruments</i> , 2021 , 5, 10	1.2	2
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36	Modeling and experimental validation of a Wf/W-fabrication by chemical vapor deposition and infiltration. <i>Nuclear Materials and Energy</i> , 2021 , 28, 101048	2.1	2
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28	Beryllium erosion and redeposition in ITER H, He and DII discharges. <i>Nuclear Fusion</i> , 2022 , 62, 036011	3.3	1
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