

Dwi Irwanto

List of Publications by Year in descending order

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56
papers

187
citations

1305906

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1336881

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58
all docs

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#	ARTICLE	IF	CITATIONS
1	Comparison of neutronic aspects in <sc>high-temperature</sc> gas-cooled reactor using <sc>ZrC</sc> and <sc>SiC</sc> Triso particle with 50 and 100 <sc>MWt</sc> power. International Journal of Energy Research, 2022, 46, 4852-4868.	2.2	2
2	Fissile utilization of uranium, thorium, and plutonium fuels for a 60 <sc>MWt</sc> block-type <sc>high-temperature</sc> gas-cooled reactor. International Journal of Energy Research, 2022, 46, 5152-5164.	2.2	0
3	Core design selection for a long-life modular gas-cooled fast reactor using <sc>OpenMC</sc> code. International Journal of Energy Research, 2022, 46, 9389-9403.	2.2	2
4	Design study of 208Pb-Bi eutectic-cooled reactor with natural uranium as fuel cycle input with radial fuel shuffling. Annals of Nuclear Energy, 2022, 171, 109003.	0.9	1
5	Enhancing the performance of a long-life modified CANDLE fast reactor by using an enriched 208Pb as coolant. Nuclear Engineering and Technology, 2021, 53, 423-429.	1.1	4
6	Reflector materials selection for core design of modular gas-cooled fast reactor using <sc>OpenMC</sc> code. International Journal of Energy Research, 2021, 45, 12071-12085.	2.2	6
7	Study on Fissile Material Usage for 50 MWt High Temperature Gas Reactor using (Pu-U)O ₂ Fuel. Journal of Physics: Conference Series, 2021, 1772, 012004.	0.3	2
8	Initial core design analysis of lead (208) -bismuth eutectic-cooled reactor with radial fuel shuffling. International Journal of Energy Research, 2021, 45, 12317-12324.	2.2	4
9	Fast Neutron Irradiation Influence Analysis in Thermal-Hydraulics Aspect of HTR-10 Reactor Using Modified PEBBLE Program. Journal of Physics: Conference Series, 2021, 1949, 012025.	0.3	0
10	Moving particle semi-implicit simulation on the molten Wood's metal downward relocation process. Nuclear Science and Techniques/Hewuli, 2021, 32, 1.	1.3	2
11	Preliminary Neutronic Analysis of 150 MWt High-Temperature Gas Reactor to Produced Electricity in Small Area. Journal of Physics: Conference Series, 2021, 2072, 012011.	0.3	0
12	Neutronic Analysis of Modular Gas-cooled Fast Reactor for 5-25% of Plutonium Fuel using Parallelization MCNP6 Code. Journal of Physics: Conference Series, 2020, 1493, 012008.	0.3	4
13	Neutronic Analysis of Lead208-Bismuth Eutectic-Cooled Modified CANDLE Reactor with Core Geometry Variations. Journal of Physics: Conference Series, 2020, 1493, 012010.	0.3	1
14	Neutronic Aspect Assessment on the use of ZrC Triso-Coated Particle (TRIZO) in a High-Temperature Gas-cooled Reactor (HTGR). Journal of Physics: Conference Series, 2020, 1493, 012029.	0.3	2
15	Neutronic analysis of comparison UN-PuN fuel and ThN fuel for 300MWth Gas Cooled Fast Reactor long life without refueling. Journal of Physics: Conference Series, 2020, 1436, 012132.	0.3	0
16	Actinide Minor Addition on Uranium Plutonium Nitride Fuel for Modular Gas Cooled Fast Reactor. Journal of Physics: Conference Series, 2020, 1493, 012020.	0.3	2
17	Study on the Effects of Enrichment and Fraction of Coated Fuel Particles on Fissile Utilization of 100 MWt Prismatic-type of High Temperature Gas Reactor. Journal of Physics: Conference Series, 2020, 1493, 012027.	0.3	2
18	Preliminary Design Study of Small Power 30-50 MWt Experiment Power Reactor based on High Temperature Pebble Bed Gas Cooled Reactor Technology. Journal of Physics: Conference Series, 2019, 1127, 012024.	0.3	6

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19	Performance of Thorium Uranium Nitride (Th, U233)N Fuel for 500 MWth GFR Long-Life Without Refuelling use FIITB-CHI Code. Journal of Physics: Conference Series, 2019, 1204, 012127.	0.3	0
20	Comparative Analysis on Fuel Breeding for Less Moderation Ratio of Water Cooled Reactor. Journal of Physics: Conference Series, 2019, 1204, 012129.	0.3	0
21	Homogenous (Th,Pu)O ₂ Fuel Utilization on High Temperature Test Reactor 30MWt. Journal of Physics: Conference Series, 2019, 1204, 012133.	0.3	0
22	HTTR 30MWth Reactor with Homogenous (Th,U) ₂ Fuel. Journal of Physics: Conference Series, 2019, 1204, 012134.	0.3	2
23	Comparison of uranium plutonium nitride (U-Pu-N) and thorium nitride (Th-N) fuel for 500MWth gas-cooled fast reactor (GFR) long life without refueling. International Journal of Energy Research, 2018, 42, 214-220.	2.2	10
24	Fuel Breeding Analysis On Low Moderated Fuel Ratio Based On Actinides Closed Water-Cooled Thorium Reactor. Journal of Physics: Conference Series, 2018, 1090, 012061.	0.3	0
25	Neutronic Comparison Study Between Pb(208)-Bi and Pb(208) as a Coolant In The Fast Reactor With Modified CANDLE Burn up Scheme.. Journal of Physics: Conference Series, 2018, 1090, 012071.	0.3	2
26	Neutronic Analysis of UN-PuN Fuel use FIITB-CHI Code for 500MWth GFR Long Life Without Refueling. Journal of Physics: Conference Series, 2018, 1090, 012033.	0.3	2
27	Comparative Evaluation on Several Reactor Type of Actinide Closed-Cycle Schemes. Journal of Physics: Conference Series, 2018, 1090, 012011.	0.3	1
28	Neutronic Study of Small Modular Reactor Thorium Based as A Model of Nuclear Power Plant for Remote Area. , 2018, , .		0
29	Charged Particle Flow Base On Mesoscale Simulation With Coupling MPCD-MD Method In Two Dimension Channel. Journal of Physics: Conference Series, 2017, 799, 012011.	0.3	2
30	Delayed Neutrons Effect on Power Reactor with Variation of Fluid Fuel Velocity at MSR Fuji-12. Journal of Physics: Conference Series, 2017, 799, 012004.	0.3	2
31	Neutronic Analysis of Thorium Nitride (Th, U233)N Fuel for 500MWth Gas Cooled Fast Reactor (GFR) Long Life without Refueling. Key Engineering Materials, 2017, 733, 47-50.	0.4	8
32	Simulation of Mechanical Stress on Stainless Steel for Pb-Bi Corrosion Test by Using ABAQUS. IOP Conference Series: Materials Science and Engineering, 2017, 180, 012009.	0.3	0
33	Fuel Fraction Analysis of 500 MWth Gas Cooled Fast Reactor with Nitride (UN-PuN) Fuel without Refueling. Journal of Physics: Conference Series, 2017, 799, 012022.	0.3	4
34	The prediction of helium gas viscosity under high pressure and high temperature with the Chapman-Enskog solution and excess viscosity. Journal of Physics: Conference Series, 2017, 799, 012008.	0.3	3
35	High temperature corrosion of cold worked YUS409D bellows of bellow-sealed valve in LBE. Journal of Physics: Conference Series, 2017, 795, 012005.	0.3	0
36	Comparative Analysis on Nuclear Fuel Sustainability Aspect of FBR. Journal of Physics: Conference Series, 2017, 799, 012006.	0.3	0

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37	Comparative Study on Various Geometrical Core Design of 300 MWth Gas Cooled Fast Reactor with UN-PuN Fuel Longlife without Refuelling. Journal of Physics: Conference Series, 2017, 877, 012064.	0.3	2
38	Comparison of Several Thermal Conductivity Constants for Thermal Hydraulic Calculation of Pebble Bed Reactor. Journal of Physics: Conference Series, 2017, 877, 012031.	0.3	0
39	International Conference on Advances in Nuclear Science and Engineering 2015. Journal of Physics: Conference Series, 2017, 799, 011001.	0.3	1
40	Preliminary Study of 20 MWth Experiment Power Reactor based on Pebble Bed Reactor. Journal of Physics: Conference Series, 2017, 877, 012058.	0.3	4
41	Prediction of Melting Penetration of Armco Iron by Liquid Uranium Using MPS_LER. Journal of Physics: Conference Series, 2017, 799, 012019.	0.3	2
42	Comparative Studies on UO ₂ Fueled HTTR Several Nuclear Data Libraries. Journal of Physics: Conference Series, 2017, 877, 012049.	0.3	0
43	Preliminary neutronic study on Pu-based OTTO cycle pebble bed reactor. Kerntechnik, 2017, 82, 643-647.	0.2	6
44	Melting Penetration Simulation of Fe-U System at High Temperature Using MPS_LER. Journal of Physics: Conference Series, 2016, 739, 012016.	0.3	0
45	Design Study of 200MWth Gas Cooled Fast Reactor with Nitride (UN-PuN) Fuel Long Life without Refueling. MATEC Web of Conferences, 2016, 82, 03008.	0.1	8
46	Preliminary Study on LiF4-ThF4-PuF4 Utilization as Fuel Salt of miniFUJI Molten Salt Reactor. Journal of Physics: Conference Series, 2016, 739, 012004.	0.3	2
47	The prospect of uranium nitride (UN-PuN) fuel for 25- 100MWe gas cooled fast reactor long life without refuelling. Journal of Physics: Conference Series, 2016, 776, 012103.	0.3	5
48	Comparative studies on plutonium utilization in HTTR with helium and carbon dioxide gas coolants. International Journal of Hydrogen Energy, 2016, 41, 6984-6989.	3.8	1
49	Preliminary Neutronic Design of High Burnup OTTO Cycle Pebble Bed Reactor. Atom Indonesia, 2015, 41, 7.	0.2	18
50	Decay heat removal without forced cooling on a small simplified PBR with an accumulative fuel loading scheme. Annals of Nuclear Energy, 2013, 60, 383-395.	0.9	9
51	A Preliminary 3D Steam Flow Analysis for CET Behavior During LSTF SBLOCA Experiment Using FLUENT Code. , 2013, , .		0
52	Burnup characteristics and fuel cycle economics of mixed uranium-thorium fuel in a simplified small pebble bed reactor. Journal of Nuclear Science and Technology, 2012, 49, 222-229.	0.7	11
53	Burnup Characteristics of a ^{235}U Fuel Loading Scheme in a 110MWt Simplified Pebble Bed Reactor. Journal of Nuclear Science and Technology, 2011, 48, 1385-1395.	0.7	14
54	Burnup Characteristics of a ^{235}U Fuel Loading Scheme in a 110 MWt Simplified Pebble Bed Reactor. Journal of Nuclear Science and Technology, 2011, 48, 1385-1395.	0.7	1

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55	Preliminary Study of Burnup Characteristics for a Simplified Small Pebble Bed Reactor. , 2010, , .		0
56	Comparison of Monte Carlo calculation methods for effective delayed neutron fraction. Annals of Nuclear Energy, 2010, 37, 1308-1315.	0.9	27