Suguru Masuzaki

List of Publications by Year in Descending Order

459 papers	7,798 citations	40 h-index	65 g-index
473 ext. papers	8,413 ext. citations	2.5 avg, IF	4.75 L-index

#	Paper	IF	Citations
459	An overview of tritium retention in dust particles from the JET-ILW divertor. *Physica Scripta*, **2022**, 97, 024008	2.6	0
458	Confinement improvement during detached phase with RMP application in deuterium plasmas of LHD. *Nuclear Fusion*, **2022**, 62, 056006	3.3	0
457	Simulation Analysis of the Carbon Deposition Profile on Directional Material Probes in the Large Helical Device Using the ERO2.0 Code. *Plasma and Fusion Research*, **2022**, 17, 2403010-2403010	0.5	0
456	Application of High-Frequency Ultrasonic Test to the Non-Destructive Inspection of W-Cu Bonded Interface. *Plasma and Fusion Research*, **2022**, 17, 2405013-2405013	0.5	0
455	Spatial Profiles of NeVI-NeX Emission in ECR-Heated Discharges of the Large Helical Device with Divertor Detachment Induced by RMP Application and Ne Impurity Seeding. *Plasma and Fusion Research*, **2022**, 17, 2402022-2402022	0.5	
454	Characteristics of plasma parameters and turbulence in the isotope-mixing and the non-mixing states in hydrogen–deuterium mixture plasmas in the large helical device. *Nuclear Fusion*, **2021**, 61, 016012	3.3	3
453	Investigation of the distribution of remaining tritium in divertor in LHD. *Nuclear Materials and Energy*, **2021**, 26, 100884	2.1	1
452	Advanced multi-step brazing for fabrication of a divertor heat removal component. *Nuclear Fusion*, **2021**, 61, 046016	3.3	3
451	Divertor heat load distribution measurements with infrared thermography in the LHD helical divertor. *Fusion Engineering and Design*, **2021**, 165, 112235	1.7	1
450	Observation of Arc Trails with Significant Damage due to Glow Discharge Wall Conditioning in the Large Helical Device. *Plasma and Fusion Research*, **2021**, 16, 1202061-1202061	0.5	
449	Investigation on tritium retention and surface properties on the first wall in the large helical Device. *Nuclear Materials and Energy*, **2021**, 27, 100906	2.1	2
448	Isotope effects on transport in LHD. *Plasma Physics and Controlled Fusion*, **2021**, 63, 094001	2	0
447	Experimental observations and modelling of radiation asymmetries during N2 seeding in LHD. *Nuclear Materials and Energy*, **2021**, 26, 100848	2.1	2
446	EUV and VUV Spectra of NeIII-NeX Line Emissions Observed in the Impurity Gas-Puffing Experiments of the Large Helical Device. *Plasma and Fusion Research*, **2021**, 16, 2402006-2402006	0.5	1
445	Application of Divertor Pumping to Long-Pulse Discharge for Particle Control in LHD. *Plasma and Fusion Research*, **2021**, 16, 1202014-1202014	0.5	0
444	Data-Driven Approach on the Mechanism of Radiative Collapse in the Large Helical Device. *Plasma and Fusion Research*, **2021**, 16, 2402010-2402010	0.5	1
443	Simulation of Impurity Transport and Deposition in the Closed Helical Divertor in the Large Helical Device. *Plasma and Fusion Research*, **2021**, 16, 2403004-2403004	0.5	1

442	Global distribution of tritium in JET with the ITER-like wall. *Nuclear Materials and Energy*, **2021**, 26, 100930	2.1	1
441	Demonstration of reduced neoclassical energy transport in Wendelstein 7-X. *Nature*, **2021**, 596, 221-226	50.4	15
440	Removal of tritium from vacuum vessel by RF heated plasmas in LHD. *Physica Scripta*, **2021**, 96, 124007	2.6	0
439	Development of the brazing technique of W and JLF-1 by Ni-P filler material. *Fusion Engineering and Design*, **2021**, 170, 112687	1.7	0
438	Quantitative evaluation of hydrogen retention of solid tin after exposure to hydrogen plasma. *Fusion Engineering and Design*, **2021**, 170, 112532	1.7	0
437	First impurity powder injection experiments in LHD. *Nuclear Materials and Energy*, **2020**, 25, 100842	2.1	5
436	Boron transport simulation using the ERO2.0 code for real-time wall conditioning in the large helical device. *Nuclear Materials and Energy*, **2020**, 25, 100853	2.1	0
435	Tritium distribution analysis of Be limiter tiles from JET-ITER like wall campaigns using imaging plate technique and □ray induced X-ray spectrometry. *Fusion Engineering and Design*, **2020**, 160, 111959	1.7	3
434	Inspection of Arc Trails Formed in Stellarator/Heliotron Devices W7-X and LHD. *Plasma and Fusion Research*, **2020**, 15, 2402012-2402012	0.5	5
433	Investigation of remaining tritium in the LHD vacuum vessel after the first deuterium experimental campaign. *Physica Scripta*, **2020**, T171, 014068	2.6	8
432	Comparison of Hydrogen Isotope Retention in Divertor Tiles of JET with the ITER-Like Wall Following Campaigns in 2011□2012 and 2015□2016. *Fusion Science and Technology*, **2020**, 76, 439-445	1.1	2
431	Application of the Advanced Multi-Step Brazing for fabrication of the high heat flux component. *Journal of Nuclear Materials*, **2020**, 538, 152264	3.3	4
430	Inspection of W 7-X plasma-facing components after the operation phase OP1.2b: observations and first assessments. *Physica Scripta*, **2020**, T171, 014033	2.6	5
429	In-vessel colorimetry of Wendelstein 7-X first wall components: variation of layer deposition distribution in OP1.2a and OP1.2b. *Physica Scripta*, **2020**, T171, 014054	2.6	3
428	Hydrogen isotope exchange at the surface of C-W mixed material layer on tungsten by gas exposure. *Fusion Engineering and Design*, **2020**, 157, 111633	1.7	1
427	Helium and hydrogen interaction in tungsten simultaneously irradiated by He+-H2+ at high temperature. *International Journal of Hydrogen Energy*, **2020**, 45, 9959-9968	6.7	4
426	Extended investigations of isotope effects on ECRH plasma in LHD. *Plasma Physics and Controlled Fusion*, **2020**, 62, 024006	2	6
425	Surface morphology of the bulk tungsten divertor tiles from JET ITER-like wall. *Physica Scripta*, **2020**, T171, 014010	2.6	3

424	Transition between Isotope-Mixing and Nonmixing States in Hydrogen-Deuterium Mixture Plasmas. *Physical Review Letters*, **2020**, 124, 025002	7.4	10
423	Divertor Detachment with Multi-Species Impurity Seeding in LHD. *Plasma and Fusion Research*, **2020**, 15, 1402051-1402051	0.5	7
422	Tritium Balance in Large Helical Device during and after the First Deuterium Plasma Experiment Campaign. *Plasma and Fusion Research*, **2020**, 15, 1405062-1405062	0.5	6
421	Prediction of Radiative Collapse in Large Helical Device Using Feature Extraction by Exhaustive Search. *Journal of Fusion Energy*, **2020**, 39, 500-511	1.6	1
420	DAMAGING OF PURE TUNGSTEN WITH DIFFERENT MICROSTRUCTURE UNDER SEQUENTIAL QSPA AND LHD PLASMA LOADS **2020**, 78-82		
419	Tritium removal from the LHD first-wall by the hydrogen plasma discharge. *Fusion Engineering and Design*, **2020**, 159, 111879	1.7	4
418	Full-torus impurity transport simulation for optimizing plasma discharge operation using a multi-species impurity powder dropper in the large helical device. *Contributions To Plasma Physics*, **2020**, 60, e201900101	1.4	4
417	Determination of retained tritium from ILW dust particles in JET. *Nuclear Materials and Energy*, **2020**, 22, 100673	2.1	2
416	Leak tight joint procedures for ODS-Cu/ODS-Cu by the advanced brazing technique. *Fusion Engineering and Design*, **2019**, 148, 111274	1.7	6
415	Impact of a resonant magnetic perturbation field on impurity radiation, divertor footprint, and core plasma transport in attached and detached plasmas in the Large Helical Device. *Nuclear Fusion*, **2019**, 59, 096009	3.3	11
414	Analysis of indefinite divertor footprint with proper orthogonal decomposition in hydrogen/deuterium plasmas in LHD. *Nuclear Materials and Energy*, **2019**, 19, 378-383	2.1	
413	Effects of drifts on divertor plasma transport in LHD. *Nuclear Materials and Energy*, **2019**, 18, 281-284	2.1	6
412	Deformation and fracture behavior of the W/ODS-Cu joint fabricated by the advanced brazing technique. *Fusion Engineering and Design*, **2019**, 146, 1733-1736	1.7	5
411	Demonstration of suppression of dust generation and partial reduction of the hydrogen retention by tungsten coated graphite divertor tiles in LHD. *Nuclear Materials and Energy*, **2019**, 18, 23-28	2.1	5
410	The isotope effect on impurities and bulk ion particle transport in the Large Helical Device. *Nuclear Fusion*, **2019**, 59, 056029	3.3	8
409	Influence of thermal shocks on the He induced surface morphology on tungsten. *Nuclear Materials and Energy*, **2019**, 18, 321-325	2.1	3
408	New installation of in-vessel Non Evaporable Getter (NEG) pumps for the divertor pump in the LHD. *Fusion Engineering and Design*, **2019**, 143, 226-232	1.7	3
407	Spectroscopic studies on the enhanced radiation with high Z rare gas seeding for mitigating divertor heat loads in LHD plasmas. *Nuclear Materials and Energy*, **2019**, 19, 195-199	2.1	1

406	Tritium distributions on W-coated divertor tiles used in the third JET ITER-like wall campaign. *Nuclear Materials and Energy*, **2019**, 18, 258-261	2.1	8
405	First divertor physics studies in Wendelstein 7-X. *Nuclear Fusion*, **2019**, 59, 096014	3.3	19
404	Analysis of mixed-material layers deposited on the toroidal array probes during the FY 2012 LHD plasma campaign. *Fusion Engineering and Design*, **2019**, 147, 111228	1.7	1
403	Overview of first Wendelstein 7-X high-performance operation. *Nuclear Fusion*, **2019**, 59, 112004	3.3	94
402	Synergistic effect of nitrogen and hydrogen seeding gases on plasma detachment in the GAMMA 10/PDX tandem mirror. *Nuclear Fusion*, **2019**, 59, 066030	3.3	9
401	New approach to the control of particle recycling using divertor pumping in the Large Helical Device. *Nuclear Fusion*, **2019**, 59, 086022	3.3	4
400	Isotope effects on energy, particle transport and turbulence in electron cyclotron resonant heating plasma of the Large Helical Device. *Nuclear Fusion*, **2019**, 59, 126040	3.3	10
399	First Application of 3D Peripheral Plasma Transport Code EMC3-EIRENE to Heliotron J. *Plasma and Fusion Research*, **2019**, 14, 3403127-3403127	0.5	2
398	Impact of additional plasma heating on detached plasma formation in divertor simulation experiments using the GAMMA 10/PDX tandem mirror. *Nuclear Materials and Energy*, **2019**, 18, 216-221	2.1	4
397	Erosion and deposition investigations on Wendelstein 7-X first wall components for the first operation phase in divertor configuration. *Fusion Engineering and Design*, **2019**, 146, 242-245	1.7	13
396	Plasma-wall interaction on the divertor tiles of JET ITER-like wall from the viewpoint of micro/nanoscopic observations. *Fusion Engineering and Design*, **2018**, 136, 199-204	1.7	4
395	Establishment of a low recycling state with full density control by active pumping of the closed helical divertor at LHD. *Nuclear Fusion*, **2018**, 58, 014005	3.3	6
394	Helium retention behavior in simultaneously He+-H2+ irradiated tungsten. *Journal of Nuclear Materials*, **2018**, 502, 289-294	3.3	12
393	Dust generation in tokamaks: Overview of beryllium and tungsten dust characterisation in JET with the ITER-like wall. *Fusion Engineering and Design*, **2018**, 136, 579-586	1.7	32
392	Deuterium retention behavior in simultaneously He+D2+ implanted tungsten. *Nuclear Materials and Energy*, **2018**, 16, 76-81	2.1	5
391	Correlation of surface chemical states with hydrogen isotope retention in divertor tiles of JET with ITER-Like Wall. *Fusion Engineering and Design*, **2018**, 132, 24-28	1.7	13
390	Investigation of dust shielding effect of intrinsic ergodic magnetic field line structures in the peripheral plasma in the large helical device. *Contributions To Plasma Physics*, **2018**, 58, 616-621	1.4	1
389	Maintainability of the helical reactor FFHR-c1 equipped with the liquid metal divertor and cartridge-type blankets. *Fusion Engineering and Design*, **2018**, 136, 1278-1285	1.7	8

388	Application of EMC3-EIRENE to Estimation of Influence of a Liquid Metal Limiter on an LHD-Type Fusion Plasma. *Plasma and Fusion Research*, **2018**, 13, 3403034-3403034	0.5	2
387	Tritium retention characteristics in dust particles in JET with ITER-like wall. *Nuclear Materials and Energy*, **2018**, 17, 279-283	2.1	15
386	Core plasma confinement during detachment transition with RMP application in LHD. *Nuclear Materials and Energy*, **2018**, 17, 137-141	2.1	6
385	Impurity transport simulation in the peripheral plasma in the large helical device with tungsten closed helical divertor. *Nuclear Materials and Energy*, **2018**, 17, 188-193	2.1	3
384	Characterized divertor footprint profile modification with the edge pressure gradient in the Large Helical Device. *Plasma Physics and Controlled Fusion*, **2018**, 60, 125001	2	3
383	Three-dimensional impurity transport modeling of neon-seeded and nitrogen-seeded LHD plasmas. *Plasma Physics and Controlled Fusion*, **2018**, 60, 084005	2	30
382	Heat loading behavior and thermomechanical analyses on plasma spray tungsten coated reduced-activation ferritic/martensitic steel. *Fusion Engineering and Design*, **2018**, 136, 1624-1628	1.7	4
381	Temperature impact on the micro structure of tungsten exposed to He irradiation in LHD. *Journal of Nuclear Materials*, **2017**, 484, 24-29	3.3	13
380	Damage and deuterium retention of re-solidified tungsten following vertical displacement event-like heat load. *Nuclear Materials and Energy*, **2017**, 12, 1303-1307	2.1	6
379	Toroidally symmetric/asymmetric effect on the divertor flux due to neon/nitrogen seeding in LHD. *Nuclear Materials and Energy*, **2017**, 12, 241-246	2.1	17
378	Initial growth phase of W-fuzz formation in ultra-long pulse helium discharge in LHD. *Nuclear Materials and Energy*, **2017**, 12, 1358-1362	2.1	7
377	Effects of modified surfaces produced at plasma-facing surface on hydrogen release behavior in the LHD. *Nuclear Materials and Energy*, **2017**, 12, 483-487	2.1	3
376	Fabrication of divertor mock-up with ODS-Cu and W by the improved brazing technique. *Nuclear Fusion*, **2017**, 57, 076009	3.3	11
375	Micro-/nano-characterization of the surface structures on the divertor tiles from JET ITER-like wall. *Fusion Engineering and Design*, **2017**, 116, 1-4	1.7	14
374	Two conceptual designs of helical fusion reactor FFHR-d1A based on ITER technologies and challenging ideas. *Nuclear Fusion*, **2017**, 57, 086046	3.3	26
373	Extension of the operational regime of the LHD towards a deuterium experiment. *Nuclear Fusion*, **2017**, 57, 102023	3.3	84
372	Conceptual design of a liquid metal limiter/divertor system for the FFHR-d1. *Fusion Engineering and Design*, **2017**, 125, 227-238	1.7	28
371	Overview of spherical tokamak research in Japan. *Nuclear Fusion*, **2017**, 57, 102005	3.3	4

370	Simulation of impurity transport in the peripheral plasma due to the emission of dust in long pulse discharges on the Large Helical Device. *Nuclear Materials and Energy*, **2017**, 12, 779-785	2.1	6
369	Investigation of heat flux deposition on divertor target on the Large Helical Device with EMC3-EIRENE modelling. *Plasma Physics and Controlled Fusion*, **2017**, 59, 085013	2	5
368	Molecular activated recombination in divertor simulation plasma on GAMMA 10/PDX. *Nuclear Materials and Energy*, **2017**, 12, 1004-1009	2.1	22
367	Influence of mixed material layer formation on hydrogen isotope and He retentions in W exposed to 2014 LHD experiment campaign. *Fusion Engineering and Design*, **2017**, 125, 458-462	1.7	2
366	Wide-range evaluation of the deposition layer thickness distribution on the first wall by reflection coefficient measurements. *Nuclear Materials and Energy*, **2017**, 12, 1219-1223	2.1	8
365	The role of the graphite divertor tiles in helium retention on the LHD wall. *Nuclear Materials and Energy*, **2017**, 13, 58-62	2.1	4
364	Recent progress of divertor simulation research using the GAMMA 10/PDX tandem mirror. *Nuclear Fusion*, **2017**, 57, 116033	3.3	43
363	Examinations for leak tightness of actively cooled components in ITER and fusion devices. *Physica Scripta*, **2017**, T170, 014045	2.6	2
362	Development of H, D, T Simultaneous TDS Measurement System and H, D, T Retention Behavior for DT Gas Exposed Tungsten Installed in LHD Plasma Campaign. *Fusion Science and Technology*, **2017**, 71, 351-356	1.1	1
361	Effects of Mild Baking on Hydrogen Removal from the Modified Surface of the First Wall in the LHD. *Plasma and Fusion Research*, **2017**, 12, 1302048-1302048	0.5	
360	Analyses of microstructure, composition and retention of hydrogen isotopes in divertor tiles of JET with the ITER-like wall. *Physica Scripta*, **2017**, T170, 014031	2.6	10
359	Preparation of erosion and deposition investigations on plasma facing components in Wendelstein 7-X. *Physica Scripta*, **2017**, T170, 014010	2.6	9
358	Tritium analysis of divertor tiles used in JET ITER-like wall campaigns by means of □ray induced x-ray spectrometry. *Physica Scripta*, **2017**, T170, 014014	2.6	4
357	Investigation of arcing on fiber-formed nanostructured tungsten by pulsed plasma during steady state plasma irradiation. *Fusion Engineering and Design*, **2016**, 112, 156-161	1.7	16
356	Design Status of the Structural Components of the Helical Fusion Reactor FFHR-d1. *Plasma and Fusion Research*, **2016**, 11, 2405061-2405061	0.5	2
355	Simulation Analysis of Carbon Deposition Profile in the Closed Helical Divertor Configuration in the Large Helical Device. *Contributions To Plasma Physics*, **2016**, 56, 651-656	1.4	4
354	Mie-Scattering Ellipsometry System for Analysis of Dust Formation Process in Large Plasma Device. *IEEE Transactions on Plasma Science*, **2016**, 44, 1032-1035	1.3	4
353	ACT2: a High Heat Flux Test Facility Using Electron Beam for Fusion Application. *Plasma and Fusion Research*, **2016**, 11, 2405089-2405089	0.5	11

352	Observation of the inward propagation of spontaneous toroidal flow from the plasma boundary in LHD. *Physics of Plasmas*, **2016**, 23, 102309	2.1	2
351	Dynamics of three-dimensional radiative structures during RMP assisted detached plasmas on the large helical device and its comparison with EMC3-EIRENE modeling. *Nuclear Fusion*, **2016**, 56, 046002	3.3	12
350	Progress of long pulse discharges by ECH in LHD. *Nuclear Fusion*, **2016**, 56, 046005	3.3	6
349	Plasma detachment study of high density helium plasmas in the Pilot-PSI device. *Nuclear Fusion*, **2016**, 56, 126006	3.3	15
348	Effect of impurity deposition layer formation on D retention in LHD plasma exposed W. *Nuclear Materials and Energy*, **2016**, 9, 84-88	2.1	8
347	Influence of carbon-dominated deposition layer on He retention and desorption in tungsten. *Fusion Engineering and Design*, **2016**, 112, 117-122	1.7	8
346	Evolution of radiation profiles during detached plasmas and radiative collapse in LHD. *Journal of Nuclear Materials*, **2015**, 463, 551-554	3.3	1
345	Novel divertor design to mitigate neutron irradiation in the helical reactor FFHR-d1. *Fusion Engineering and Design*, **2015**, 98-99, 1629-1633	1.7	10
344	Studies of dust transport in long pulse plasma discharges in the large helical device. *Nuclear Fusion*, **2015**, 55, 053014	3.3	18
343	Development of impurity seeding and radiation enhancement in the helical divertor of LHD. *Nuclear Fusion*, **2015**, 55, 083016	3.3	19
342	Overview of transport and MHD stability study: focusing on the impact of magnetic field topology in the Large Helical Device. *Nuclear Fusion*, **2015**, 55, 104018	3.3	7
341	Radiated power distributions in impurity-seeded plasmas in LHD. *Journal of Nuclear Materials*, **2015**, 463, 640-643	3.3	6
340	Global helium particle balance in LHD. *Journal of Nuclear Materials*, **2015**, 463, 1080-1083	3.3	15
339	Multi-pin Langmuir probe measurement for identification of blob propagation characteristics in the Large Helical Device. *Journal of Nuclear Materials*, **2015**, 463, 761-764	3.3	6
338	Effect of neutral hydrogen on edge impurity behavior in stochastic magnetic field layer of Large Helical Device. *Journal of Nuclear Materials*, **2015**, 463, 644-648	3.3	4
337	Vacuum ultraviolet spectroscopy in detached plasmas with impurity gas seeding in LHD. *Journal of Nuclear Materials*, **2015**, 463, 561-564	3.3	2
336	Progress of divertor simulation research toward the realization of detached plasma using a large tandem mirror device. *Journal of Nuclear Materials*, **2015**, 463, 537-540	3.3	16
335	Potential of Copper Alloys using a Divertor Heat Sink in the Helical Reactor FFHR-d1 and their Brazing Properties with Tungsten Armor by using the Typical Candidate Filler Materials. *Plasma and Fusion Research*, **2015**, 10, 3405035-3405035	0.5	11

334	Preliminary Examination of Reflection Coefficient Measurement of RGB Lights on the First Wall in LHD. *Plasma and Fusion Research*, **2015**, 10, 1202074-1202074	0.5	3
333	3D effects of edge magnetic field configuration on divertor/scrape-off layer transport and optimization possibilities for a future reactor. *Nuclear Fusion*, **2015**, 55, 104021	3.3	10
332	Real-time mass measurement of dust particles deposited on vessel wall in a divertor simulator using quartz crystal microbalances. *Journal of Nuclear Materials*, **2015**, 463, 865-868	3.3	0
331	Analysis of the three-dimensional trajectories of dusts observed with a stereoscopic fast framing camera in the Large Helical Device. *Journal of Nuclear Materials*, **2015**, 463, 861-864	3.3	6
330	Effect of the RF wall conditioning on the high performance plasmas in the Large Helical Device. *Journal of Nuclear Materials*, **2015**, 463, 1100-1103	3.3	10
329	Impact of 3D magnetic field structure on boundary and divertor plasmas in stellarator/heliotron devices. *Journal of Nuclear Materials*, **2015**, 463, 2-10	3.3	6
328	Plasma wall interaction in long-pulse helium discharge in LHD Microscopic modification of the wall surface and its impact on particle balance and impurity generation. *Journal of Nuclear Materials* , **2015**, 463, 91-98	3.3	28
327	Analysis of radiation environment at divertor in helical reactor FFHR-d1. *Fusion Engineering and Design*, **2014**, 89, 1939-1943	1.7	16
326	Comparison of hydrogen isotope retention for tungsten probes in LHD vacuum vessel during the experimental campaigns in 2011 and 2012. *Fusion Engineering and Design*, **2014**, 89, 1091-1095	1.7	7
325	Measured and Modeled Radiation Profiles for LHD Plasma -´A Comparison during Plasma Detachment. *Plasma and Fusion Research*, **2014**, 9, 3402064-3402064	0.5	3
324	Simulation Analysis of Dust-Particle Transport in the Peripheral Plasma in the Large Helical Device. *Plasma and Fusion Research*, **2014**, 9, 3403132-3403132	0.5	2
323	Contribution of H2plasma etching to radial profile of amount of dust particles in a divertor simulator. *Journal of Physics: Conference Series*, **2014**, 518, 012009	0.3	
322	Two-dimensional study of edge impurity transport in the Large Helical Device. *Plasma Physics and Controlled Fusion*, **2014**, 56, 094007	2	13
321	First EMC3-EIRENE Simulations with Divertor Legs of LHD in Realistic Device Geometry. *Contributions To Plasma Physics*, **2014**, 54, 437-441	1.4	32
320	A New Deduction Method of Heat Flux Evolution From Thermal Probe Data. *Contributions To Plasma Physics*, **2014**, 54, 285-290	1.4	1
319	Mitigation of large amplitude edge-localized modes by resonant magnetic perturbations on LHD. *Nuclear Fusion*, **2014**, 54, 033001	3.3	4
318	Design of structural components for the helical reactor FFHR-d1A. *Fusion Engineering and Design*, **2014**, 89, 2336-2340	1.7	12
317	Kinetic effect of high energy ions on the temperature profile in the boundary plasma region. *Journal of Nuclear Materials*, **2013**, 438, S472-S474	3.3	2

316	Evaluation of the surface morphologies and erosion/deposition profiles on the LHD first-wall by using toroidal array probes. *Journal of Nuclear Materials*, **2013**, 442, S873-S879	3.3	6
315	Recent results of divertor simulation research using an end-cell of a large tandem mirror device. *Journal of Nuclear Materials*, **2013**, 438, S738-S741	3.3	23
314	Study of hydrogen isotopes behavior in carbon based materials with in situ ion beam analysis under plasma exposure. *Journal of Nuclear Materials*, **2013**, 438, S1036-S1039	3.3	4
313	Control of 3D edge radiation structure with resonant magnetic perturbation fields applied to the stochastic layer and stabilization of radiative divertor plasma in LHD. *Nuclear Fusion*, **2013**, 53, 093032	3.3	33
312	Influence of the resonant magnetic perturbations on transport in the Large Helical Device. *Nuclear Fusion*, **2013**, 53, 113012	3.3	9
311	Radial-build design and support system for the helical DEMO reactor FFHR-d1. *Fusion Engineering and Design*, **2013**, 88, 2033-2037	1.7	7
310	Intermittent transport in edge plasma with a 3-D magnetic geometry in the Large Helical Device. *Journal of Nuclear Materials*, **2013**, 438, S563-S566	3.3	2
309	Edge and divertor plasma measurements with ion sensitive and Mach probes in LHD. *Journal of Nuclear Materials*, **2013**, 438, S1228-S1231	3.3	3
308	Enhancement of hydrogen isotope retention in tungsten exposed to LHD plasmas. *Journal of Nuclear Materials*, **2013**, 438, S1055-S1058	3.3	8
307	Removal of carbon deposited film and hydrogen retention control by low temperature HCN reactive plasmas. *Journal of Nuclear Materials*, **2013**, 438, S1092-S1095	3.3	2
306	Discharge power dependence of carbon dust flux in a divertor simulator. *Journal of Nuclear Materials*, **2013**, 438, S788-S791	3.3	4
305	Development of high-grade VPS-tungsten coatings on F82H reduced activation steel. *Journal of Nuclear Materials*, **2013**, 442, S287-S291	3.3	7
304	Microstructural characterization of mixed-material deposition layer on the LHD divertor tiles by using nano-geological diagnosis. *Journal of Nuclear Materials*, **2013**, 438, S818-S821	3.3	14
303	Divertor heat and particle control experiments on the large helical device. *Journal of Nuclear Materials*, **2013**, 438, S133-S138	3.3	13
302	Effect of a baffle divertor structure on neutral hydrogen and helium transport in the Large Helical Device. *Journal of Nuclear Materials*, **2013**, 438, S559-S562	3.3	2
301	Enhancement of hydrogen isotope retention capacity for the impurity deposited tungsten by long-term plasma exposure in LHD. *Fusion Engineering and Design*, **2013**, 88, 1699-1703	1.7	12
300	Effects of DC substrate bias voltage on dust flux in the Large Helical Device. *Journal of Nuclear Materials*, **2013**, 438, S727-S730	3.3	3
299	Directional Material Probe for Deposition Layer Studies. *Plasma and Fusion Research*, **2013**, 8, 1202110-1202110	0.7	2

298	Impact of arcing on carbon and tungsten: from the observations in JT-60U, LHD and NAGDIS-II. *Nuclear Fusion*, **2013**, 53, 053013	3.3	32
297	Flux Control of Carbon Nanoparticles Generated due to Interactions between Hydrogen Plasmas and Graphite Using DC-Biased Substrates. *Japanese Journal of Applied Physics*, **2013**, 52, 11NA08	1.4	1
296	Initial experiments towards edge plasma control with a closed helical divertor in LHD. *Nuclear Fusion*, **2013**, 53, 063014	3.3	25
295	Modification of the helical divertor footprint induced by the net toroidal plasma current in the LHD heliotron. *Nuclear Fusion*, **2013**, 53, 112001	3.3	4
294	Transition of poloidal viscosity by electrode biasing in the Large Helical Device. *Nuclear Fusion*, **2013**, 53, 073014	3.3	5
293	Edge impurity transport study in the stochastic layer of LHD and the scrape-off layer of HL-2A. *Nuclear Fusion*, **2013**, 53, 033011	3.3	29
292	Effective screening of iron impurities in the ergodic layer of the Large Helical Device with a metallic first wall. *Nuclear Fusion*, **2013**, 53, 093017	3.3	34
291	Extension of operation regimes and investigation of three-dimensional currentless plasmas in the Large Helical Device. *Nuclear Fusion*, **2013**, 53, 104015	3.3	32
290	Steady-state operation using a dipole mode ion cyclotron heating antenna and 77 GHz electron cyclotron heating in the Large Helical Device. *Nuclear Fusion*, **2013**, 53, 063017	3.3	21
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