

Mohmmadrezaa Nematollahi

List of Publications by Year in descending order

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16
papers

129
citations

1307594

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1199594

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16
all docs

16
docs citations

16
times ranked

112
citing authors

#	ARTICLE	IF	CITATIONS
1	Development of a hybrid method to assess grid-related LOOP scenarios for an NPP. Reliability Engineering and System Safety, 2021, 206, 107298.	8.9	4
2	Conceptual design of a high neutron flux research reactor core with low enriched uranium fuel and low plutonium production. Nuclear Engineering and Technology, 2020, 52, 499-507.	2.3	4
3	Comparison of semi-heavy water and H ₂ O as coolant for a conceptual research reactor from the view point of neutronic parameters. Progress in Nuclear Energy, 2020, 118, 103126.	2.9	1
4	Innovative designed fuel pin of conceptual semi-heavy water research reactor with improved reactor safety. Progress in Nuclear Energy, 2020, 130, 103559.	2.9	0
5	2D PIV study of flow accelerated corrosion downstream a typical industrial gate valve. Progress in Nuclear Energy, 2020, 121, 103260.	2.9	2
6	Using MCNP for success criteria evaluation of reactor trip function in probabilistic modeling of ATWS upon LOOP. Progress in Nuclear Energy, 2018, 106, 270-277.	2.9	2
7	Numerical study of mass transfer coefficient in a T-junction. International Journal of Hydrogen Energy, 2016, 41, 7027-7035.	7.1	2
8	CFX study of flow accelerated corrosion via mass transfer coefficient calculation in a double elbow. International Journal of Hydrogen Energy, 2016, 41, 7036-7046.	7.1	5
9	Dose assessment of radionuclides dispersion from Bushehr nuclear power plant stack under normal operation and accident conditions. International Journal of Hydrogen Energy, 2015, 40, 15198-15205.	7.1	23
10	Experimental and numerical void fraction measurement for modeled two-phase flow inside a vertical pipe. Annals of Nuclear Energy, 2015, 83, 188-192.	1.8	9
11	Reduction of core damage frequency via new design for emergency core cooling system in a typical PWR. International Journal of Hydrogen Energy, 2015, 40, 15185-15191.	7.1	0
12	Void fraction measurement in modeled two-phase flow inside a vertical pipe by using polyethylene phantoms. International Journal of Hydrogen Energy, 2015, 40, 15206-15212.	7.1	18
13	Sensitivity analysis on the effect of software-induced common cause failure probability in the computer-based reactor trip system unavailability. Annals of Nuclear Energy, 2013, 57, 294-303.	1.8	18
14	Evaluating the core damage frequency of a TRIGA research reactor using risk assessment tool software. Nuclear Engineering and Design, 2011, 241, 2942-2947.	1.7	7
15	Coupling CFAST fire modeling and SAPHIRE probabilistic assessment software for internal fire safety evaluation of a typical TRIGA research reactor. Reliability Engineering and System Safety, 2010, 95, 166-172.	8.9	12
16	Development and application of a Risk Assessment Tool. Reliability Engineering and System Safety, 2008, 93, 1130-1137.	8.9	22