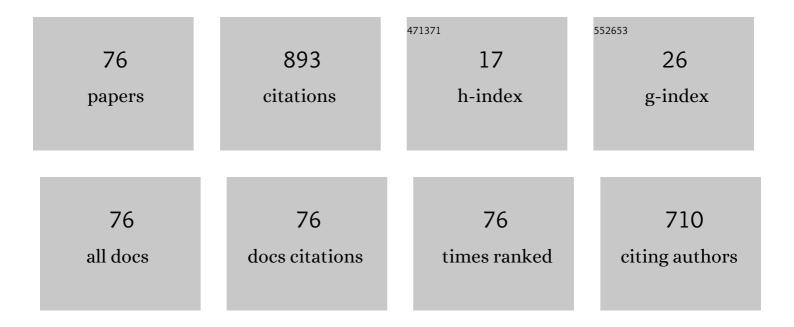
Abdolhamid Minuchehr

List of Publications by Year in descending order

Source: https://exaly.com/author-pdf/6860053/publications.pdf Version: 2024-02-01



| # | Article | IF | CITATIONS |
|----|--|-----|-----------|
| 1 | A pure dynamic Monte Carlo code for the neutronic analysis of nuclear reactors. Annals of Nuclear Energy, 2022, 165, 108627. | 0.9 | 3 |
| 2 | A novel adaptive mesh free approach for even- parity neutron transport equation. Annals of Nuclear Energy, 2022, 175, 109253. | 0.9 | 0 |
| 3 | Modeling and experimental validation of the steady-state counteractive facilitated transport of Th(IV) and hydrogen ions through hollow-fiber renewal liquid membrane. Chemical Papers, 2021, 75, 325-336. | 1.0 | 2 |
| 4 | A mesh-free treatment for even parity neutron transport equation. Annals of Nuclear Energy, 2021, 158, 108292. | 0.9 | 1 |
| 5 | Primary radiation damage in silicon from the viewpoint of a machine learning interatomic potential. Physical Review Materials, 2021, 5, . | 0.9 | 3 |
| 6 | A goal-oriented and self-adaptive mesh refinement approach for the even parity neutron transport equation. Progress in Nuclear Energy, 2020, 119, 102996. | 1.3 | 1 |
| 7 | Insights into the primary radiation damage of silicon by a machine learning interatomic potential. Materials Research Letters, 2020, 8, 364-372. | 4.1 | 15 |
| 8 | A coupled neutronic/thermal–hydraulic module for the transient analysis of VVER-1000 reactor during reactivity insertion accidents. Progress in Nuclear Energy, 2020, 121, 103249. | 1.3 | 4 |
| 9 | Numerical study of hyperstoichiometric fuel creep (UO2+) in fuel clad interaction of WWER1000. Annals of Nuclear Energy, 2019, 133, 950-959. | 0.9 | 2 |
| 10 | Economic and Efficient phosphonic functional groups mesoporous silica for uranium selective adsorption from aqueous solutions. Scientific Reports, 2019, 9, 9686. | 1.6 | 24 |
| 11 | Laplace transform finite volume modeling of water hammer along fluid–structure interaction. Computers and Mathematics With Applications, 2019, 77, 2821-2832. | 1.4 | 5 |
| 12 | Mechanical properties of carbon nanotube- and graphene-reinforced Araldite LY/Aradur HY 5052 resin epoxy composites: a molecular dynamics study. Journal of Molecular Modeling, 2019, 25, 191. | 0.8 | 12 |
| 13 | Implementation and comparison of different prompt and delayed α-static approaches. Progress in Nuclear Energy, 2019, 114, 210-226. | 1.3 | 3 |
| 14 | The role of chromium and nickel on the thermal and mechanical properties of FeNiCr austenitic stainless steels under high pressure and temperature: a molecular dynamics study. Molecular Simulation, 2019, 45, 672-684. | 0.9 | 8 |
| 15 | Even-parity Boltzmann transport equation applied for response (contributon) flux calculation based on the spatial channel theory. Computers and Mathematics With Applications, 2018, 75, 4378-4396. | 1.4 | 1 |
| 16 | An improved convergence rate for the prompt α eigenvalue calculation in α-k iteration methods. Annals of Nuclear Energy, 2018, 118, 15-25. | 0.9 | 3 |
| 17 | Th(<scp>iv</scp>) recovery from aqueous waste via hollow fiber renewal liquid membrane (HFRLM) in recycling mode: modelling and experimental validation. RSC Advances, 2017, 7, 7413-7423. | 1.7 | 19 |
| 18 | A semi-analytical treatment of xenon oscillations. Annals of Nuclear Energy, 2017, 106, 127-135. | 0.9 | 4 |

| # | Article | IF | CITATIONS |
|----|---|-----|-----------|
| 19 | An improved combinatorial geometry model for arbitrary geometry in DSMC. Journal of Statistical Mechanics: Theory and Experiment, 2017, 2017, 033203. | 0.9 | 2 |
| 20 | ENTRANS: A platform for finite elements modeling of 3D neutron transport equation, Part II. Multidimensional implementation. Annals of Nuclear Energy, 2017, 101, 534-551. | 0.9 | 11 |
| 21 | Dehydration of acetonitrile using cross-linked sodium alginate membrane containing nano-sized NaA zeolite. Chemical Papers, 2017, 71, 1143-1153. | 1.0 | 9 |
| 22 | Synthesis of enhanced phosphonic functional groups mesoporous silica for uranium selective adsorption from aqueous solutions. Scientific Reports, 2017, 7, 11675. | 1.6 | 57 |
| 23 | An arbitrary geometry finite element method for the adjoint neutron transport equation. Annals of Nuclear Energy, 2017, 110, 511-525. | 0.9 | 5 |
| 24 | Accurate reflective boundary condition in the P method. Annals of Nuclear Energy, 2017, 109, 583-599. | 0.9 | 0 |
| 25 | Thorium pertraction through hollow fiber renewal liquid membrane (HFRLM) using Cyanex 272 as carrier. Progress in Nuclear Energy, 2017, 100, 209-220. | 1.3 | 12 |
| 26 | Electromagnetism Mechanism for Enhancing the Refueling Cycle Length of a WWER-1000. Nuclear Engineering and Technology, 2017, 49, 43-53. | 1.1 | 1 |
| 27 | Synchronized forward-adjoint neutron transport using method of characteristics. Progress in Nuclear Energy, 2016, 92, 211-219. | 1.3 | 4 |
| 28 | Th(IV) transport from nitrate media through hollow fiber renewal liquid membrane. Journal of Membrane Science, 2016, 520, 374-384. | 4.1 | 15 |
| 29 | The development of GPU-based parallel PRNG for Monte Carlo applications in CUDA Fortran. AIP Advances, 2016, 6, . | 0.6 | 4 |
| 30 | Space independent xenon oscillations control in WER reactor: A bifurcation analysis approach. Progress in Nuclear Energy, 2016, 88, 19-27. | 1.3 | 8 |
| 31 | The thermo-mechanical properties estimation of fullerene-reinforced resin epoxy composites by molecular dynamics simulation $\hat{a} \in \mathcal{C}$ A comparative study. Polymer, 2016, 88, 9-18. | 1.8 | 81 |
| 32 | Analysis of control rod drop accident in PWRs with multipoint kinetics method. Annals of Nuclear Energy, 2016, 88, 194-203. | 0.9 | 5 |
| 33 | DgSMC-B code: A robust and autonomous direct simulation Monte Carlo code for arbitrary geometries. AIP Advances, 2016, 6, 075208. | 0.6 | 1 |
| 34 | An approach to stability analysis of spatial xenon oscillations in WWER-1000 reactors. Annals of Nuclear Energy, 2015, 79, 125-132. | 0.9 | 16 |
| 35 | Development and validation of a new multigroup Monte Carlo Criticality Calculations (MC3) code. Progress in Nuclear Energy, 2015, 81, 53-59. | 1.3 | 7 |
| 36 | An adaptive node refinement for particle transport techniques based on response matrix and collision probability methods. Annals of Nuclear Energy, 2015, 80, 248-253. | 0.9 | 2 |

| # | Article | IF | CITATIONS |
|----|--|-----|-----------|
| 37 | Hybrid space–angle adaptivity for whole-core particle transport calculations. Annals of Nuclear Energy, 2015, 80, 254-260. | 0.9 | 1 |
| 38 | Numerical study of fuel–clad mechanical interaction during long-term burnup of WWER1000. Annals of Nuclear Energy, 2015, 80, 267-278. | 0.9 | 13 |
| 39 | A novel optimization method, Effective Discrete Firefly Algorithm, for fuel reload design of nuclear reactors. Annals of Nuclear Energy, 2015, 81, 263-275. | 0.9 | 13 |
| 40 | Spatially adaptive hp refinement approach for PN neutron transport equation using spectral element method. Annals of Nuclear Energy, 2015, 85, 1066-1076. | 0.9 | 10 |
| 41 | Resonance self-shielding calculation using subgroup method and ABC algorithm. Progress in Nuclear Energy, 2015, 78, 303-309. | 1.3 | 1 |
| 42 | Three-dimensional high order nodal code, ACNECH, for the neutronic modeling of hexagonal-z geometry. Annals of Nuclear Energy, 2014, 68, 172-182. | 0.9 | 6 |
| 43 | Short-term and long-term analysis of WWER-1000 containment parameters in a large break LOCA. Progress in Nuclear Energy, 2014, 74, 201-212. | 1.3 | 11 |
| 44 | Analysis of thermal–hydraulic parameters of WWER-1000 containment in a large break LOCA. Annals of Nuclear Energy, 2014, 68, 101-111. | 0.9 | 11 |
| 45 | Bat algorithm for the fuel arrangement optimization of reactor core. Annals of Nuclear Energy, 2014, 64, 144-151. | 0.9 | 42 |
| 46 | Coupled neutronic/thermo-hydraulic analysis of water/Al2O3 nanofluids in a VVER-1000 reactor. Annals of Nuclear Energy, 2014, 65, 72-77. | 0.9 | 27 |
| 47 | Full scope thermal-neutronic analysis of LOFA in a WWER-1000 reactor core by coupling PARCS v2.7 and COBRA-EN. Progress in Nuclear Energy, 2014, 74, 193-200. | 1.3 | 15 |
| 48 | Simulation of rod ejection accident in a WWER-1000 Nuclear Reactor by using PARCS code. Annals of Nuclear Energy, 2014, 65, 132-140. | 0.9 | 9 |
| 49 | An adaptive p-refinement strategy applied to nodal expansion method in 3D Cartesian geometry. Annals of Nuclear Energy, 2014, 63, 575-586. | 0.9 | 1 |
| 50 | Heterogeneous reactor core transport technique using response matrix and collision probability methods. Annals of Nuclear Energy, 2013, 62, 137-143. | 0.9 | 10 |
| 51 | Development of a high order and multi-dimensional nodal code, ACNEC3D, for reactor core analysis. Annals of Nuclear Energy, 2013, 55, 211-224. | 0.9 | 21 |
| 52 | Evaluation of compressible flow in spherical fueled reactors using the porous media model. Annals of Nuclear Energy, 2013, 57, 185-194. | 0.9 | 8 |
| 53 | An adaptive mesh refinement approach for average current nodal expansion method in 2-D rectangular geometry. Annals of Nuclear Energy, 2013, 55, 61-70. | 0.9 | 10 |
| 54 | Advanced progressive real coded genetic algorithm for nuclear system availability optimization through preventive maintenance scheduling. Annals of Nuclear Energy, 2013, 60, 64-72. | 0.9 | 18 |

Abdolhamid Minuchehr

| # | Article | IF | CITATIONS |
|----|--|-----|-----------|
| 55 | Self-adaptive global best harmony search algorithm applied to reactor core fuel management optimization. Annals of Nuclear Energy, 2013, 62, 86-102. | 0.9 | 17 |
| 56 | Nuclear reactor core optimization with Parallel Integer Coded Genetic Algorithm. Annals of Nuclear Energy, 2013, 60, 308-315. | 0.9 | 20 |
| 57 | Coupled hp-adaptivity for average current nodal expansion method in 2-D rectangular geometry. Progress in Nuclear Energy, 2013, 66, 25-34. | 1.3 | 5 |
| 58 | FDBACE code for fast Doppler broadening ACE format based on parallel processing. Annals of Nuclear Energy, 2013, 62, 170-177. | 0.9 | 3 |
| 59 | Investigation of PWR core optimization using harmony search algorithms. Annals of Nuclear Energy, 2013, 57, 1-15. | 0.9 | 27 |
| 60 | Differential harmony search algorithm to optimize PWRs loading pattern. Nuclear Engineering and Design, 2013, 257, 161-174. | 0.8 | 25 |
| 61 | Multi-objective loading pattern enhancement of PWR based on the Discrete Firefly Algorithm. Annals of Nuclear Energy, 2013, 57, 151-163. | 0.9 | 32 |
| 62 | Continuous firefly algorithm applied to PWR core pattern enhancement. Nuclear Engineering and Design, 2013, 258, 107-115. | 0.8 | 39 |
| 63 | WWER core pattern enhancement using adaptive improved harmony search. Nuclear Engineering and Design, 2013, 254, 23-32. | 0.8 | 21 |
| 64 | PWR loading pattern optimization using Harmony Search algorithm. Annals of Nuclear Energy, 2013, 53, 288-298. | 0.9 | 24 |
| 65 | A NOVEL APPROACH TO FIND OPTIMIZED NEUTRON ENERGY GROUP STRUCTURE IN MOX THERMAL LATTICES USING SWARM INTELLIGENCE. Nuclear Engineering and Technology, 2013, 45, 951-960. | 1.1 | 7 |
| 66 | Impact assessment of upscattering on resonance calculation using improved ultrafine energy group method. Annals of Nuclear Energy, 2012, 49, 114-121. | 0.9 | 2 |
| 67 | Transient analysis of break below the grid in Tehran research reactor using the newly enhanced COBRA-EN code. Annals of Nuclear Energy, 2012, 49, 1-11. | 0.9 | 5 |
| 68 | Bayesian inference along Markov Chain Monte Carlo approach for PWR core loading pattern optimization. Annals of Nuclear Energy, 2012, 50, 150-157. | 0.9 | 2 |
| 69 | Coupled neutronic thermal–hydraulic transient analysis of accidents in PWRs. Annals of Nuclear Energy, 2012, 50, 158-166. | 0.9 | 15 |
| 70 | Performance comparison of zeroth order nodal expansion methods in 3D rectangular geometry. Nuclear Engineering and Design, 2012, 252, 248-266. | 0.8 | 19 |
| 71 | An investigation for an optimized neutron energy-group structure in thermal lattices using Particle Swarm Optimization. Annals of Nuclear Energy, 2012, 47, 53-61. | 0.9 | 12 |
| 72 | A PN-based approach along PSO scheme for PWR core reloading patterns optimization. Nuclear Engineering and Design, 2012, 248, 206-215. | 0.8 | 5 |

| # | Article | IF | CITATIONS |
|----|--|-----|-----------|
| 73 | An advanced method for determination of loss of coolant accident in nuclear power plants. Nuclear Engineering and Design, 2011, 241, 2013-2019. | 0.8 | 2 |
| 74 | An adaptive finite element approach for neutron transport equation. Nuclear Engineering and Design, 2011, 241, 2143-2154. | 0.8 | 20 |
| 75 | 226Ra concentration in the teeth of habitants of areas with high level of natural radioactivity in Ramsar. Journal of Environmental Radioactivity, 2006, 89, 212-218. | 0.9 | 14 |
| 76 | Readout responses of inclined strips in position-sensitive detectors. IEEE Transactions on Nuclear Science, 2001, 48, 2321-2323. | 1.2 | 0 |