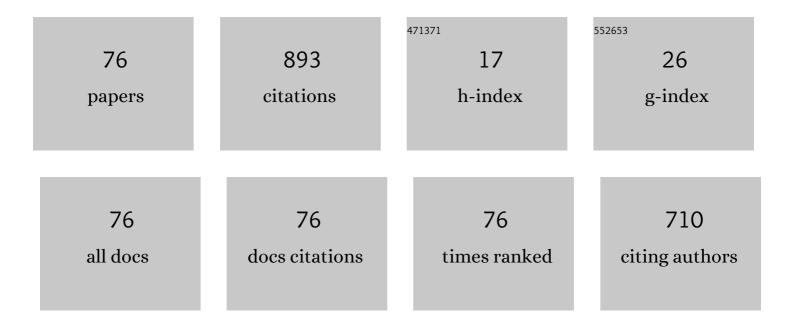
Abdolhamid Minuchehr

List of Publications by Year in descending order

Source: https://exaly.com/author-pdf/6860053/publications.pdf Version: 2024-02-01



#	Article	IF	CITATIONS
1	A pure dynamic Monte Carlo code for the neutronic analysis of nuclear reactors. Annals of Nuclear Energy, 2022, 165, 108627.	0.9	3
2	A novel adaptive mesh free approach for even- parity neutron transport equation. Annals of Nuclear Energy, 2022, 175, 109253.	0.9	0
3	Modeling and experimental validation of the steady-state counteractive facilitated transport of Th(IV) and hydrogen ions through hollow-fiber renewal liquid membrane. Chemical Papers, 2021, 75, 325-336.	1.0	2
4	A mesh-free treatment for even parity neutron transport equation. Annals of Nuclear Energy, 2021, 158, 108292.	0.9	1
5	Primary radiation damage in silicon from the viewpoint of a machine learning interatomic potential. Physical Review Materials, 2021, 5, .	0.9	3
6	A goal-oriented and self-adaptive mesh refinement approach for the even parity neutron transport equation. Progress in Nuclear Energy, 2020, 119, 102996.	1.3	1
7	Insights into the primary radiation damage of silicon by a machine learning interatomic potential. Materials Research Letters, 2020, 8, 364-372.	4.1	15
8	A coupled neutronic/thermal–hydraulic module for the transient analysis of VVER-1000 reactor during reactivity insertion accidents. Progress in Nuclear Energy, 2020, 121, 103249.	1.3	4
9	Numerical study of hyperstoichiometric fuel creep (UO2+) in fuel clad interaction of WWER1000. Annals of Nuclear Energy, 2019, 133, 950-959.	0.9	2
10	Economic and Efficient phosphonic functional groups mesoporous silica for uranium selective adsorption from aqueous solutions. Scientific Reports, 2019, 9, 9686.	1.6	24
11	Laplace transform finite volume modeling of water hammer along fluid–structure interaction. Computers and Mathematics With Applications, 2019, 77, 2821-2832.	1.4	5
12	Mechanical properties of carbon nanotube- and graphene-reinforced Araldite LY/Aradur HY 5052 resin epoxy composites: a molecular dynamics study. Journal of Molecular Modeling, 2019, 25, 191.	0.8	12
13	Implementation and comparison of different prompt and delayed α-static approaches. Progress in Nuclear Energy, 2019, 114, 210-226.	1.3	3
14	The role of chromium and nickel on the thermal and mechanical properties of FeNiCr austenitic stainless steels under high pressure and temperature: a molecular dynamics study. Molecular Simulation, 2019, 45, 672-684.	0.9	8
15	Even-parity Boltzmann transport equation applied for response (contributon) flux calculation based on the spatial channel theory. Computers and Mathematics With Applications, 2018, 75, 4378-4396.	1.4	1
16	An improved convergence rate for the prompt α eigenvalue calculation in α-k iteration methods. Annals of Nuclear Energy, 2018, 118, 15-25.	0.9	3
17	Th(<scp>iv</scp>) recovery from aqueous waste via hollow fiber renewal liquid membrane (HFRLM) in recycling mode: modelling and experimental validation. RSC Advances, 2017, 7, 7413-7423.	1.7	19
18	A semi-analytical treatment of xenon oscillations. Annals of Nuclear Energy, 2017, 106, 127-135.	0.9	4

#	Article	IF	CITATIONS
19	An improved combinatorial geometry model for arbitrary geometry in DSMC. Journal of Statistical Mechanics: Theory and Experiment, 2017, 2017, 033203.	0.9	2
20	ENTRANS: A platform for finite elements modeling of 3D neutron transport equation, Part II. Multidimensional implementation. Annals of Nuclear Energy, 2017, 101, 534-551.	0.9	11
21	Dehydration of acetonitrile using cross-linked sodium alginate membrane containing nano-sized NaA zeolite. Chemical Papers, 2017, 71, 1143-1153.	1.0	9
22	Synthesis of enhanced phosphonic functional groups mesoporous silica for uranium selective adsorption from aqueous solutions. Scientific Reports, 2017, 7, 11675.	1.6	57
23	An arbitrary geometry finite element method for the adjoint neutron transport equation. Annals of Nuclear Energy, 2017, 110, 511-525.	0.9	5
24	Accurate reflective boundary condition in the P method. Annals of Nuclear Energy, 2017, 109, 583-599.	0.9	0
25	Thorium pertraction through hollow fiber renewal liquid membrane (HFRLM) using Cyanex 272 as carrier. Progress in Nuclear Energy, 2017, 100, 209-220.	1.3	12
26	Electromagnetism Mechanism for Enhancing the Refueling Cycle Length of a WWER-1000. Nuclear Engineering and Technology, 2017, 49, 43-53.	1.1	1
27	Synchronized forward-adjoint neutron transport using method of characteristics. Progress in Nuclear Energy, 2016, 92, 211-219.	1.3	4
28	Th(IV) transport from nitrate media through hollow fiber renewal liquid membrane. Journal of Membrane Science, 2016, 520, 374-384.	4.1	15
29	The development of GPU-based parallel PRNG for Monte Carlo applications in CUDA Fortran. AIP Advances, 2016, 6, .	0.6	4
30	Space independent xenon oscillations control in WER reactor: A bifurcation analysis approach. Progress in Nuclear Energy, 2016, 88, 19-27.	1.3	8
31	The thermo-mechanical properties estimation of fullerene-reinforced resin epoxy composites by molecular dynamics simulation $\hat{a} \in \mathcal{C}$ A comparative study. Polymer, 2016, 88, 9-18.	1.8	81
32	Analysis of control rod drop accident in PWRs with multipoint kinetics method. Annals of Nuclear Energy, 2016, 88, 194-203.	0.9	5
33	DgSMC-B code: A robust and autonomous direct simulation Monte Carlo code for arbitrary geometries. AIP Advances, 2016, 6, 075208.	0.6	1
34	An approach to stability analysis of spatial xenon oscillations in WWER-1000 reactors. Annals of Nuclear Energy, 2015, 79, 125-132.	0.9	16
35	Development and validation of a new multigroup Monte Carlo Criticality Calculations (MC3) code. Progress in Nuclear Energy, 2015, 81, 53-59.	1.3	7
36	An adaptive node refinement for particle transport techniques based on response matrix and collision probability methods. Annals of Nuclear Energy, 2015, 80, 248-253.	0.9	2

#	Article	IF	CITATIONS
37	Hybrid space–angle adaptivity for whole-core particle transport calculations. Annals of Nuclear Energy, 2015, 80, 254-260.	0.9	1
38	Numerical study of fuel–clad mechanical interaction during long-term burnup of WWER1000. Annals of Nuclear Energy, 2015, 80, 267-278.	0.9	13
39	A novel optimization method, Effective Discrete Firefly Algorithm, for fuel reload design of nuclear reactors. Annals of Nuclear Energy, 2015, 81, 263-275.	0.9	13
40	Spatially adaptive hp refinement approach for PN neutron transport equation using spectral element method. Annals of Nuclear Energy, 2015, 85, 1066-1076.	0.9	10
41	Resonance self-shielding calculation using subgroup method and ABC algorithm. Progress in Nuclear Energy, 2015, 78, 303-309.	1.3	1
42	Three-dimensional high order nodal code, ACNECH, for the neutronic modeling of hexagonal-z geometry. Annals of Nuclear Energy, 2014, 68, 172-182.	0.9	6
43	Short-term and long-term analysis of WWER-1000 containment parameters in a large break LOCA. Progress in Nuclear Energy, 2014, 74, 201-212.	1.3	11
44	Analysis of thermal–hydraulic parameters of WWER-1000 containment in a large break LOCA. Annals of Nuclear Energy, 2014, 68, 101-111.	0.9	11
45	Bat algorithm for the fuel arrangement optimization of reactor core. Annals of Nuclear Energy, 2014, 64, 144-151.	0.9	42
46	Coupled neutronic/thermo-hydraulic analysis of water/Al2O3 nanofluids in a VVER-1000 reactor. Annals of Nuclear Energy, 2014, 65, 72-77.	0.9	27
47	Full scope thermal-neutronic analysis of LOFA in a WWER-1000 reactor core by coupling PARCS v2.7 and COBRA-EN. Progress in Nuclear Energy, 2014, 74, 193-200.	1.3	15
48	Simulation of rod ejection accident in a WWER-1000 Nuclear Reactor by using PARCS code. Annals of Nuclear Energy, 2014, 65, 132-140.	0.9	9
49	An adaptive p-refinement strategy applied to nodal expansion method in 3D Cartesian geometry. Annals of Nuclear Energy, 2014, 63, 575-586.	0.9	1
50	Heterogeneous reactor core transport technique using response matrix and collision probability methods. Annals of Nuclear Energy, 2013, 62, 137-143.	0.9	10
51	Development of a high order and multi-dimensional nodal code, ACNEC3D, for reactor core analysis. Annals of Nuclear Energy, 2013, 55, 211-224.	0.9	21
52	Evaluation of compressible flow in spherical fueled reactors using the porous media model. Annals of Nuclear Energy, 2013, 57, 185-194.	0.9	8
53	An adaptive mesh refinement approach for average current nodal expansion method in 2-D rectangular geometry. Annals of Nuclear Energy, 2013, 55, 61-70.	0.9	10
54	Advanced progressive real coded genetic algorithm for nuclear system availability optimization through preventive maintenance scheduling. Annals of Nuclear Energy, 2013, 60, 64-72.	0.9	18

Abdolhamid Minuchehr

#	Article	IF	CITATIONS
55	Self-adaptive global best harmony search algorithm applied to reactor core fuel management optimization. Annals of Nuclear Energy, 2013, 62, 86-102.	0.9	17
56	Nuclear reactor core optimization with Parallel Integer Coded Genetic Algorithm. Annals of Nuclear Energy, 2013, 60, 308-315.	0.9	20
57	Coupled hp-adaptivity for average current nodal expansion method in 2-D rectangular geometry. Progress in Nuclear Energy, 2013, 66, 25-34.	1.3	5
58	FDBACE code for fast Doppler broadening ACE format based on parallel processing. Annals of Nuclear Energy, 2013, 62, 170-177.	0.9	3
59	Investigation of PWR core optimization using harmony search algorithms. Annals of Nuclear Energy, 2013, 57, 1-15.	0.9	27
60	Differential harmony search algorithm to optimize PWRs loading pattern. Nuclear Engineering and Design, 2013, 257, 161-174.	0.8	25
61	Multi-objective loading pattern enhancement of PWR based on the Discrete Firefly Algorithm. Annals of Nuclear Energy, 2013, 57, 151-163.	0.9	32
62	Continuous firefly algorithm applied to PWR core pattern enhancement. Nuclear Engineering and Design, 2013, 258, 107-115.	0.8	39
63	WWER core pattern enhancement using adaptive improved harmony search. Nuclear Engineering and Design, 2013, 254, 23-32.	0.8	21
64	PWR loading pattern optimization using Harmony Search algorithm. Annals of Nuclear Energy, 2013, 53, 288-298.	0.9	24
65	A NOVEL APPROACH TO FIND OPTIMIZED NEUTRON ENERGY GROUP STRUCTURE IN MOX THERMAL LATTICES USING SWARM INTELLIGENCE. Nuclear Engineering and Technology, 2013, 45, 951-960.	1.1	7
66	Impact assessment of upscattering on resonance calculation using improved ultrafine energy group method. Annals of Nuclear Energy, 2012, 49, 114-121.	0.9	2
67	Transient analysis of break below the grid in Tehran research reactor using the newly enhanced COBRA-EN code. Annals of Nuclear Energy, 2012, 49, 1-11.	0.9	5
68	Bayesian inference along Markov Chain Monte Carlo approach for PWR core loading pattern optimization. Annals of Nuclear Energy, 2012, 50, 150-157.	0.9	2
69	Coupled neutronic thermal–hydraulic transient analysis of accidents in PWRs. Annals of Nuclear Energy, 2012, 50, 158-166.	0.9	15
70	Performance comparison of zeroth order nodal expansion methods in 3D rectangular geometry. Nuclear Engineering and Design, 2012, 252, 248-266.	0.8	19
71	An investigation for an optimized neutron energy-group structure in thermal lattices using Particle Swarm Optimization. Annals of Nuclear Energy, 2012, 47, 53-61.	0.9	12
72	A PN-based approach along PSO scheme for PWR core reloading patterns optimization. Nuclear Engineering and Design, 2012, 248, 206-215.	0.8	5

#	Article	IF	CITATIONS
73	An advanced method for determination of loss of coolant accident in nuclear power plants. Nuclear Engineering and Design, 2011, 241, 2013-2019.	0.8	2
74	An adaptive finite element approach for neutron transport equation. Nuclear Engineering and Design, 2011, 241, 2143-2154.	0.8	20
75	226Ra concentration in the teeth of habitants of areas with high level of natural radioactivity in Ramsar. Journal of Environmental Radioactivity, 2006, 89, 212-218.	0.9	14
76	Readout responses of inclined strips in position-sensitive detectors. IEEE Transactions on Nuclear Science, 2001, 48, 2321-2323.	1.2	0