

Jianzhu Cao

List of Publications by Year in descending order

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14
papers

90
citations

1684188

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h-index

1372567

10
g-index

14
all docs

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docs citations

14
times ranked

49
citing authors

#	ARTICLE	IF	CITATIONS
1	Analysis of Fission Productsâ€™ Release in Pebble-Bed High-Temperature Gas-Cooled Reactor Fuel Elements Using a Modified FRESCO II Numerical Model. Science and Technology of Nuclear Installations, 2021, 2021, 1-8.	0.8	0
2	Multi-scenario validation of CALMET-RIMPUFF for local-scale atmospheric dispersion modeling around a nuclear powerplant site with complex topography. Journal of Environmental Radioactivity, 2021, 229-230, 106547.	1.7	2
3	BULK DIFFUSION OF HYDROGEN AND TRITIUM WITH 2.25CR1MO STEEL IN CHINESE 200 MW HTR-PM FROM FIRST-PRINCIPLE. The Proceedings of the International Conference on Nuclear Engineering (ICONE), 2019, 2019.27, 1930.	0.0	0
4	Study of tritium in the primary loop of HTR-10: Experiment and theoretical calculations. Progress in Nuclear Energy, 2018, 105, 99-105.	2.9	16
5	Design of the Online Gross D_2O Monitoring Instrument at the Exit of the Helium Purification System in HTR-PM. Science and Technology of Nuclear Installations, 2018, 2018, 1-12.	0.8	1
6	Source Term Analysis of Tritium in HTR-10. Fusion Science and Technology, 2017, 71, 671-678.	1.1	14
7	Experimental research on the radioactive dust in the primary loop of HTR-10. Nuclear Engineering and Design, 2017, 324, 372-378.	1.7	24
8	Source Term Study on Tritium in HTR-PM: Theoretical Calculations and Experimental Design. Science and Technology of Nuclear Installations, 2017, 2017, 1-11.	0.8	11
9	Diffusion Behaviors of Hydrogen Isotopes in Incoloy 800H: A First-Principles Study. Science and Technology of Nuclear Installations, 2017, 2017, 1-6.	0.8	2
10	Adsorption Behaviors of Cobalt on the Graphite and SiC Surface: A First-Principles Study. Science and Technology of Nuclear Installations, 2017, 2017, 1-8.	0.8	3
11	Radiation Protection Practices during the Helium Circulator Maintenance of the 10â€™MW High Temperature Gas-Cooled Reactor-Test Module (HTR-10). Science and Technology of Nuclear Installations, 2016, 2016, 1-13.	0.8	4
12	ICONE23-1479 THE RESEARCH PROGRESS ON THE RADIOACTIVE GRAPHITE DUST IN HTR-10. The Proceedings of the International Conference on Nuclear Engineering (ICONE), 2015, 2015.23, _ICONE23-1-_ICONE23-1.	0.0	2
13	ICONE23-1697 STUDY ON THE PRODUCTION MECHANISM OF CO-60 IN THE PRIMARY LOOP OF HTR-10. The Proceedings of the International Conference on Nuclear Engineering (ICONE), 2015, 2015.23, _ICONE23-1-_ICONE23-1.	0.0	2
14	The design and thermohydraulics study of the HTR-10 High Temperature Helium Experimental Loop. Progress in Nuclear Energy, 2014, 77, 329-335.	2.9	9