Tara M Pandya

List of Publications by Year in descending order

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1684188 1125743 14 167 5 13 citations g-index h-index papers 25 25 25 169 all docs docs citations times ranked citing authors

#	Article	lF	CITATIONS
1	Validation of Light Water Reactor Ex-Core Calculations with VERA. Nuclear Technology, 2022, 208, 794-810.	1.2	1
2	Two-step neutronics calculations with Shift and Griffin for advanced reactor systems. Annals of Nuclear Energy, 2022, 173, 109131.	1.8	5
3	Secondary-Source Core Reload Modeling with VERA. Nuclear Science and Engineering, 2021, 195, 320-337.	1.1	4
4	HIGH-FIDELITY EX-CORE CAPABILITIES IN VERA. EPJ Web of Conferences, 2021, 247, 06024.	0.3	2
5	EFFECT OF FISSION SOURCE SPECTRUM ON MONTE CARLO CALCULATION OF EX-CORE QUANTITIES. EPJ Web of Conferences, 2021, 247, 02027.	0.3	3
6	Optimization of processor allocation for domain decomposed Monte Carlo calculations. Parallel Computing, 2019, 87, 77-86.	2.1	6
7	Eigenvalue Solvers for Modeling Nuclear Reactors on Leadership Class Machines. Nuclear Science and Engineering, 2018, 190, 31-44.	1.1	8
8	Nuclide depletion capabilities in the Shift Monte Carlo code. Annals of Nuclear Energy, 2018, 114, 259-276.	1.8	26
9	Deterministically estimated fission source distributions for Monte Carlo k-eigenvalue problems. Annals of Nuclear Energy, 2018, 119, 7-22.	1.8	1
10	Hot zero power reactor calculations using the Insilico code. Journal of Computational Physics, 2016, 314, 700-711.	3.8	6
11	Implementation, capabilities, and benchmarking of Shift, a massively parallel Monte Carlo radiation transport code. Journal of Computational Physics, 2016, 308, 239-272.	3.8	79
12	Massively Parallel, Three-Dimensional Transport Solutions for the <i>k</i> -Eigenvalue Problem. Nuclear Science and Engineering, 2014, 177, 111-125.	1.1	15
13	Scalable Parallel Prefix Solvers for Discrete Ordinates Transport in Multidimensions. Nuclear Science and Engineering, 2011, 169, 245-261.	1.1	1
14	Proposed Design and Operation of a Heat Pipe Reactor using the Sandia National Laboratories Annular Core Test Facility and Existing UZrH Fuel Pins. AIP Conference Proceedings, 2005, , .	0.4	5