

# Laurence Kh Leung

## List of Publications by Year in descending order

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17  
papers

784  
citations

840776

11  
h-index

888059

17  
g-index

17  
all docs

17  
docs citations

17  
times ranked

356  
citing authors

#	ARTICLE	IF	CITATIONS
1	A mechanistic bubble crowding model for predicting critical heat flux in subchannels of a bundle. <i>Annals of Nuclear Energy</i> , 2020, 137, 107085.	1.8	13
2	A phenomenological CHF model for mixing-vane spacers in a subchannel of a rod bundle. <i>Annals of Nuclear Energy</i> , 2020, 142, 107445.	1.8	4
3	Numerical investigation of buoyancy effect on heat transfer to carbon dioxide flow in a tube at supercritical pressures. <i>International Journal of Heat and Mass Transfer</i> , 2018, 117, 595-606.	4.8	36
4	A review on recent heat transfer studies to supercritical pressure water in channels. <i>Applied Thermal Engineering</i> , 2018, 142, 573-596.	6.0	84
5	A predictive-corrective process for predicting forced convective heat transfer in heated tubes at supercritical pressures. <i>International Journal of Heat and Mass Transfer</i> , 2017, 110, 374-382.	4.8	3
6	Improvement of buoyancy and acceleration parameters for forced and mixed convective heat transfer to supercritical fluids flowing in vertical tubes. <i>International Journal of Heat and Mass Transfer</i> , 2017, 106, 1144-1156.	4.8	107
7	Nonuniform heat transfer of supercritical water in a tight rod bundle – Assessment of correlations. <i>Annals of Nuclear Energy</i> , 2017, 110, 570-583.	1.8	16
8	Heat transfer from a 2 Å– 2 wire-wrapped rod bundle to supercritical pressure water. <i>International Journal of Heat and Mass Transfer</i> , 2016, 97, 486-501.	4.8	41
9	Overview of methods to increase dryout power in CANDU fuel bundles. <i>Nuclear Engineering and Design</i> , 2015, 287, 131-138.	1.7	5
10	Review of R&D for supercritical water cooled reactors. <i>Progress in Nuclear Energy</i> , 2014, 77, 282-299.	2.9	41
11	Experimental investigation of heat transfer from a 2 Å–2 rod bundle to supercritical pressure water. <i>Nuclear Engineering and Design</i> , 2014, 275, 205-218.	1.7	68
12	Experimental investigation of heat transfer for supercritical pressure water flowing in vertical annular channels. <i>Nuclear Engineering and Design</i> , 2011, 241, 4045-4054.	1.7	50
13	The 2006 CHF look-up table. <i>Nuclear Engineering and Design</i> , 2007, 237, 1909-1922.	1.7	272
14	Pressure drops for steam and water flow in heated tubes. <i>Nuclear Engineering and Design</i> , 2005, 235, 53-65.	1.7	22
15	Prediction of the obstacle effect on film-boiling heat transfer. <i>Nuclear Engineering and Design</i> , 2005, 235, 687-700.	1.7	6
16	A model for predicting diabatic pressure drops in multi-element fuel channels. <i>Nuclear Engineering and Design</i> , 1989, 110, 299-312.	1.7	5
17	Two-phase pressure drop through obstructions. <i>Nuclear Engineering and Design</i> , 1988, 105, 349-361.	1.7	11