Laurence Kh Leung

List of Publications by Year
in descending order

17 papers	784 citations	840776 11 h-index	888059 17 g-index
17 all docs	17 docs citations	17 times ranked	356 citing authors

#	Article	IF	Citations
1	The 2006 CHF look-up table. Nuclear Engineering and Design, 2007, 237, 1909-1922.	1.7	272
2	Improvement of buoyancy and acceleration parameters for forced and mixed convective heat transfer to supercritical fluids flowing in vertical tubes. International Journal of Heat and Mass Transfer, 2017, 106, 1144-1156.	4.8	107
3	A review on recent heat transfer studies to supercritical pressure water in channels. Applied Thermal Engineering, 2018, 142, 573-596.	6.0	84
4	Experimental investigation of heat transfer from a 2Ã—2 rod bundle to supercritical pressure water. Nuclear Engineering and Design, 2014, 275, 205-218.	1.7	68
5	Experimental investigation of heat transfer for supercritical pressure water flowing in vertical annular channels. Nuclear Engineering and Design, 2011, 241, 4045-4054.	1.7	50
6	Review of R&D for supercritical water cooled reactors. Progress in Nuclear Energy, 2014, 77, 282-299.	2.9	41
7	Heat transfer from a 2 Ã— 2 wire-wrapped rod bundle to supercritical pressure water. International Journal of Heat and Mass Transfer, 2016, 97, 486-501.	4.8	41
8	Numerical investigation of buoyancy effect on heat transfer to carbon dioxide flow in a tube at supercritical pressures. International Journal of Heat and Mass Transfer, 2018, 117, 595-606.	4.8	36
9	Pressure drops for steam and water flow in heated tubes. Nuclear Engineering and Design, 2005, 235, 53-65.	1.7	22
10	Nonuniform heat transfer of supercritical water in a tight rod bundle â€“ Assessment of correlations. Annals of Nuclear Energy, 2017, 110, 570-583.	1.8	16
11	A mechanistic bubble crowding model for predicting critical heat flux in subchannels of a bundle. Annals of Nuclear Energy, 2020, 137, 107085.	1.8	13
12	Two-phase pressure drop through obstructions. Nuclear Engineering and Design, 1988, 105, 349-361.	1.7	11
13	Prediction of the obstacle effect on film-boiling heat transfer. Nuclear Engineering and Design, 2005, 235, 687-700.	1.7	6
14	A model for predicting diabatic pressure drops in multi-element fuel channels. Nuclear Engineering and Design, 1989, 110, 299-312.	1.7	5
15	Overview of methods to increase dryout power in CANDU fuel bundles. Nuclear Engineering and Design, 2015, 287, 131-138.	1.7	5
16	A phenomenological CHF model for mixing-vane spacers in a subchannel of a rod bundle. Annals of Nuclear Energy, 2020, 142, 107445.	1.8	4
17	A predictive-corrective process for predicting forced convective heat transfer in heated tubes at supercritical pressures. International Journal of Heat and Mass Transfer, 2017, 110, 374-382.	4.8	3