

Jan Dufek

List of Publications by Year in descending order

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Version: 2024-02-01

27
papers

335
citations

1040056

9
h-index

839539

18
g-index

27
all docs

27
docs citations

27
times ranked

137
citing authors

| # | ARTICLE | IF | CITATIONS |
|----|--|-----|-----------|
| 1 | The stochastic implicit Euler method – A stable coupling scheme for Monte Carlo burnup calculations. <i>Annals of Nuclear Energy</i> , 2013, 60, 295-300. | 1.8 | 42 |
| 2 | Stochastic Approximation for Monte Carlo Calculation of Steady-State Conditions in Thermal Reactors. <i>Nuclear Science and Engineering</i> , 2006, 152, 274-283. | 1.1 | 41 |
| 3 | Numerical Stability of Existing Monte Carlo Burnup Codes in Cycle Calculations of Critical Reactors. <i>Nuclear Science and Engineering</i> , 2009, 162, 307-311. | 1.1 | 41 |
| 4 | Numerical stability of the predictor–corrector method in Monte Carlo burnup calculations of critical reactors. <i>Annals of Nuclear Energy</i> , 2013, 56, 34-38. | 1.8 | 37 |
| 5 | Preventing xenon oscillations in Monte Carlo burnup calculations by enforcing equilibrium xenon distribution. <i>Annals of Nuclear Energy</i> , 2013, 60, 78-85. | 1.8 | 31 |
| 6 | Fission matrix based Monte Carlo criticality calculations. <i>Annals of Nuclear Energy</i> , 2009, 36, 1270-1275. | 1.8 | 26 |
| 7 | Building the nodal nuclear data dependences in a many-dimensional state-variable space. <i>Annals of Nuclear Energy</i> , 2011, 38, 1569-1577. | 1.8 | 12 |
| 8 | Description of a stable scheme for steady-state coupled Monte Carlo–thermal–hydraulic calculations. <i>Annals of Nuclear Energy</i> , 2014, 68, 1-3. | 1.8 | 12 |
| 9 | Stability and convergence problems of the Monte Carlo fission matrix acceleration methods. <i>Annals of Nuclear Energy</i> , 2009, 36, 1648-1651. | 1.8 | 11 |
| 10 | Neutron batch size optimisation methodology for Monte Carlo criticality calculations. <i>Annals of Nuclear Energy</i> , 2015, 75, 620-626. | 1.8 | 11 |
| 11 | Development and comparison of spectral methods for passive acoustic anomaly detection in nuclear power plants. <i>Applied Acoustics</i> , 2014, 83, 100-107. | 3.3 | 9 |
| 12 | Estimation of errors in the cumulative Monte Carlo fission source. <i>Annals of Nuclear Energy</i> , 2014, 72, 151-155. | 1.8 | 8 |
| 13 | Monte Carlo criticality calculations accelerated by a growing neutron population. <i>Annals of Nuclear Energy</i> , 2016, 94, 16-21. | 1.8 | 8 |
| 14 | Optimal neutron population growth in accelerated Monte Carlo criticality calculations. <i>Annals of Nuclear Energy</i> , 2018, 117, 297-304. | 1.8 | 8 |
| 15 | Derivation of a stable coupling scheme for Monte Carlo burnup calculations with the thermal–hydraulic feedback. <i>Annals of Nuclear Energy</i> , 2013, 62, 260-263. | 1.8 | 7 |
| 16 | Time step length versus efficiency of Monte Carlo burnup calculations. <i>Annals of Nuclear Energy</i> , 2014, 72, 409-412. | 1.8 | 6 |
| 17 | Stabilization effect of fission source in coupled Monte Carlo simulations. <i>Nuclear Engineering and Technology</i> , 2017, 49, 1095-1099. | 2.3 | 6 |
| 18 | Stochastic-deterministic response matrix method for reactor transients. <i>Annals of Nuclear Energy</i> , 2020, 140, 107103. | 1.8 | 5 |

| # | ARTICLE | IF | CITATIONS |
|----|--|-----|-----------|
| 19 | From once-through nuclear fuel cycle to accelerator-driven transmutation. Nuclear Instruments and Methods in Physics Research, Section A: Accelerators, Spectrometers, Detectors and Associated Equipment, 2006, 562, 630-633. | 1.6 | 4 |
| 20 | Correlation of errors in the Monte Carlo fission source and the fission matrix fundamental-mode eigenvector. Annals of Nuclear Energy, 2016, 94, 415-421. | 1.8 | 4 |
| 21 | Does neutron clustering affect tally errors in Monte Carlo criticality calculations?. Annals of Nuclear Energy, 2021, 155, 108130. | 1.8 | 3 |
| 22 | Optimal time step length and statistics in Monte Carlo burnup simulations. Annals of Nuclear Energy, 2020, 139, 107244. | 1.8 | 2 |
| 23 | An efficient parallel computing scheme for Monte Carlo criticality calculations. Annals of Nuclear Energy, 2009, 36, 1276-1279. | 1.8 | 1 |
| 24 | Performance of the Explicit Euler and Predictor-Corrector-Based Coupling Schemes in Monte Carlo Burnup Calculations of Fast Reactors. Nuclear Technology, 2015, 191, 193-198. | 1.2 | 0 |
| 25 | APPLICATION OF RESPONSE MATRIX METHOD TO TRANSIENT SIMULATIONS OF NUCLEAR SYSTEMS. EPJ Web of Conferences, 2021, 247, 04014. | 0.3 | 0 |
| 26 | OPTIMISATION OF MONTE CARLO BURNUP SIMULATIONS. EPJ Web of Conferences, 2021, 247, 04016. | 0.3 | 0 |
| 27 | Optimised Iteration in Coupled Monte Carlo Thermal-Hydraulics Calculations. , 2014, , . | | 0 |