

Sebastijan Brezinsek

List of Publications by Year in descending order

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531
papers

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30070
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#	ARTICLE	IF	CITATIONS
1	Progress from ASDEX Upgrade experiments in preparing the physics basis of ITER operation and DEMO scenario development. Nuclear Fusion, 2022, 62, 042006.	3.5	15
2	Latest results of Eurofusion plasma-facing components research in the areas of power loading, material erosion and fuel retention. Nuclear Fusion, 2022, 62, 042013.	3.5	11
3	Operating a full tungsten actively cooled tokamak: overview of WEST first phase of operation. Nuclear Fusion, 2022, 62, 042007.	3.5	39
4	Investigation of plasma wall interactions between tungsten plasma facing components and helium plasmas in the WEST tokamak. Nuclear Fusion, 2022, 62, 076028.	3.5	22
5	Overview of the COMPASS results [*] . Nuclear Fusion, 2022, 62, 042021.	3.5	7
6	Experimental confirmation of efficient island divertor operation and successful neoclassical transport optimization in Wendelstein 7-X. Nuclear Fusion, 2022, 62, 042022.	3.5	24
7	Plasma-surface interaction in the stellarator W7-X: conclusions drawn from operation with graphite plasma-facing components. Nuclear Fusion, 2022, 62, 016006.	3.5	12
8	Simulating energetic particle losses in JET plasmas with a reverse integration biasing scheme. Nuclear Fusion, 2022, 62, 026026.	3.5	3
9	Overview of JET results for optimising ITER operation. Nuclear Fusion, 2022, 62, 042026.	3.5	52
10	Beryllium erosion and redeposition in ITER H, He and D-T discharges. Nuclear Fusion, 2022, 62, 036011.	3.5	13
11	Predictions of radiation pattern and in-out asymmetries in the DEMO scrape-off layer using fluid neutrals. Nuclear Fusion, 2022, 62, 056015.	3.5	4
12	Analysis of hydrogen fueling, recycling, and confinement at Wendelstein 7-X via a single-reservoir particle balance. Nuclear Fusion, 2022, 62, 036023.	3.5	5
13	Predictive 3D modelling of erosion and deposition in ITER with ERO2.0: from beryllium main wall, tungsten divertor to full-tungsten device. Physica Scripta, 2022, 97, 014001.	2.5	2
14	Investigation of boron distribution and material migration on the W7-X divertor by picosecond LIBS. Physica Scripta, 2022, 97, 024005.	2.5	0
15	EUROfusion-theory and advanced simulation coordination (E-TASC): programme and the role of high performance computing. Plasma Physics and Controlled Fusion, 2022, 64, 034005.	2.1	2
16	Fluid, kinetic and hybrid approaches for neutral and trace ion edge transport modelling in fusion devices. Nuclear Fusion, 2022, 62, 086051.	3.5	13
17	Isotope removal experiment in JET-ILW in view of T-removal after the 2nd DT campaign at JET. Physica Scripta, 2022, 97, 044001.	2.5	7
18	Simulation Analysis of the Carbon Deposition Profile on Directional Material Probes in the Large Helical Device Using the ERO2.0 Code. Plasma and Fusion Research, 2022, 17, 2403010-2403010.	0.7	1

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19	Overview of the emissivity measurements performed in WEST: in situ and post-mortem observations. Nuclear Fusion, 2022, 62, 096023.	3.5	11
20	Enhanced performance in fusion plasmas through turbulence suppression by megaelectronvolt ions. Nature Physics, 2022, 18, 776-782.	16.7	36
21	Measuring gross beryllium erosion with visible cameras in JET. Nuclear Fusion, 2022, 62, 126001.	3.5	2
22	Effect of magnetic geometry on the energy partition between ions and electrons in the scrape-off layer of magnetic fusion devices. Nuclear Fusion, 2022, 62, 094002.	3.5	2
23	Analyses of deuterium retention in tungsten and graphite first wall materials by laser-induced ablation spectroscopy on EAST. Fusion Engineering and Design, 2021, 162, 112108.	1.9	8
24	The upgraded TOMAS device: A toroidal plasma facility for wall conditioning, plasma production, and plasma-surface interaction studies. Review of Scientific Instruments, 2021, 92, 023506.	1.3	13
25	Simulation of Impurity Transport and Deposition in the Closed Helical Divertor in the Large Helical Device. Plasma and Fusion Research, 2021, 16, 2403004-2403004.	0.7	2
26	Monitoring Removal of W Layer from Ag Substrate Using Balmer- α Emission of Backscattered Hydrogen Atoms in Low Density Gas Discharge. Acta Physica Polonica A 138, 643 (2020), ERRATUM. Acta Physica Polonica A, 2021, 139, 170-171.	0.5	0
27	The impact of ELM mitigation on tungsten source in the EAST divertor. Nuclear Fusion, 2021, 61, 046046.	3.5	4
28	A sensitivity analysis of numerical predictions for beryllium erosion and migration in ITER. Nuclear Materials and Energy, 2021, 26, 100904.	1.3	9
29	Deuterium and helium outgassing following plasma discharges in WEST: Delayed D outgassing during D-to-He changeover experiments studied with threshold ionization mass spectrometry. Nuclear Materials and Energy, 2021, 26, 100885.	1.3	5
30	Hydrogen content in divertor baffle tiles in Wendelstein 7-X. Nuclear Materials and Energy, 2021, 26, 100943.	1.3	7
31	Symmetries of ^{13}C tracer deposition in EAST D and He plasmas investigated on the sub-mm to 100 mm scale by deuteron nuclear reaction analysis. Fusion Engineering and Design, 2021, 166, 112292.	1.9	1
32	Impurity behaviour in JET-ILW plasmas fuelled with gas and/or with pellets: a comparative study with the transport code COREDIV. Nuclear Fusion, 2021, 61, 066027.	3.5	1
33	Data on erosion and hydrogen fuel retention in Beryllium plasma-facing materials. Nuclear Materials and Energy, 2021, 27, 100994.	1.3	21
34	An interpretive model for the double peaks of divertor tungsten erosion during type-I ELMs in EAST. Nuclear Fusion, 2021, 61, 086011.	3.5	7
35	Cross diagnostics measurements of heat load profiles on the lower tungsten divertor of WEST in L-mode experiments. Nuclear Materials and Energy, 2021, 27, 100961.	1.3	10
36	Use of the Culham He model He II atomic data in JET EDGE2D-EIRENE simulations. Nuclear Materials and Energy, 2021, 27, 101010.	1.3	0

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37	Progress on MATEO probe heads and observation system. Fusion Engineering and Design, 2021, 167, 112297.	1.9	1
38	The impact of surface morphology on the erosion of metallic surfaces – Modelling with the 3D Monte-Carlo code ERO2.0. Nuclear Materials and Energy, 2021, 27, 100987.	1.3	9
39	Zeeman-resolved TDLAS using metastable levels of Ar in the weakly magnetized plasma of the linear plasma device PSI-2. Journal Physics D: Applied Physics, 2021, 54, 395001.	2.8	2
40	In situ study of short-term retention of deuterium in tungsten during and after plasma exposure in PSI-2. Nuclear Fusion, 2021, 61, 096006.	3.5	5
41	Overview of the results from divertor experiments with attached and detached plasmas at Wendelstein 7-X and their implications for steady-state operation. Nuclear Fusion, 2021, 61, 106003.	3.5	24
42	Demonstration of reduced neoclassical energy transport in Wendelstein 7-X. Nature, 2021, 596, 221-226.	27.8	69
43	Divertor power loads and scrape off layer width in the large aspect ratio full tungsten tokamak WEST. Nuclear Fusion, 2021, 61, 096027.	3.5	17
44	Characterization of injection and confinement improvement through impurity induced profile modifications on the Wendelstein 7-X stellarator. Physics of Plasmas, 2021, 28, .	1.9	18
45	Deposition of ¹³ C tracer and impurity elements on the divertor of Wendelstein 7-X. Physica Scripta, 2021, 96, 124023.	2.5	3
46	Combined high fluence and high cycle number transient loading of ITER-like monoblocks in Magnum-PSI. Nuclear Fusion, 2021, 61, 116045.	3.5	9
47	Characterization of neutral particle fluxes from ICWC and ECWC plasmas in the TOMAS facility. Physica Scripta, 2021, 96, 124025.	2.5	7
48	Gross and net erosion balance of plasma-facing materials in full-W tokamaks. Nuclear Fusion, 2021, 61, 116006.	3.5	13
49	Interpretative modeling of impurity transport and tungsten sources in WEST boundary plasma. Nuclear Fusion, 2021, 61, 126015.	3.5	4
50	Modelling of tungsten contamination and screening in WEST plasma discharges. Nuclear Fusion, 2021, 61, 106019.	3.5	13
51	Deuterium retention in MeV ion-irradiated beryllium. Journal of Nuclear Materials, 2021, 555, 153139.	2.7	5
52	Effectiveness of local methane and hydrogen injection into the scrape-off layer of W7-X by means of the multi-purpose manipulator. Fusion Engineering and Design, 2021, 173, 112786.	1.9	1
53	Measurements of the energy distribution of W atoms sputtered by low energy Ar ions using high-resolution Doppler spectroscopy. Plasma Physics and Controlled Fusion, 2021, 63, 015008.	2.1	4
54	Monitoring of tritium and impurities in the first wall of fusion devices using a LIBS based diagnostic. Nuclear Fusion, 2021, 61, 125001.	3.5	31

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55	Quantification of erosion pattern using picosecond-LIBS on a vertical divertor target element exposed in W7-X. Nuclear Fusion, 2021, 61, 016025.	3.5	14
56	Double pulse laser-induced breakdown spectroscopy for the analysis of plasma-facing components. Physica Scripta, 2021, 96, 124064.	2.5	5
57	Sustained W-melting experiments on actively cooled ITER-like plasma facing unit in WEST. Physica Scripta, 2021, 96, 124057.	2.5	19
58	Plasma-wall interaction studies in W7-X: main results from the recent divertor operations. Physica Scripta, 2021, 96, 124059.	2.5	10
59	Erosion of tungsten marker layers in W7-X. Physica Scripta, 2021, 96, 124070.	2.5	6
60	Short-term retention in metallic PFCs: modelling in view of mass spectrometry and LIBS. Physica Scripta, 2021, 96, 124079.	2.5	0
61	Investigation of laser ablation features of molybdenum bulk for picosecond laser-based techniques in fusion devices. Fusion Engineering and Design, 2020, 151, 111379.	1.9	15
62	Impact of divertor configuration on recycling neutral fluxes for ITER-like wall in JET H-mode plasmas. Plasma Physics and Controlled Fusion, 2020, 62, 035006.	2.1	8
63	Wall conditioning in fusion devices with superconducting coils. Plasma Physics and Controlled Fusion, 2020, 62, 034002.	2.1	25
64	The role of hydrogen molecular effects on detachment in Magnum-PSI. Physics of Plasmas, 2020, 27, .	1.9	16
65	3-Dimensional analysis of layer structured samples with high depth resolution using picosecond laser-induced breakdown spectroscopy. Applied Surface Science, 2020, 532, 147185.	6.1	15
66	Highly depth-resolved characterization of fusion-related tungsten material based on picosecond laser-induced breakdown spectroscopy. Journal of Analytical Atomic Spectrometry, 2020, 35, 2867-2879.	3.0	14
67	Micro-structured tungsten, a high heat flux pulse proof material. Nuclear Materials and Energy, 2020, 25, 100789.	1.3	2
68	Laser-induced ablation of tantalum in a wide range of pulse durations. Applied Physics A: Materials Science and Processing, 2020, 126, 1.	2.3	6
69	¹³ C tracer deposition in EAST D and He plasmas investigated by high-throughput deuteron nuclear reaction analysis mapping. Nuclear Materials and Energy, 2020, 25, 100805.	1.3	7
70	Integrated modelling: Coupling of surface evolution and plasma-impurity transport. Nuclear Materials and Energy, 2020, 25, 100821.	1.3	7
71	The role of drifts on the isotope effect on divertor plasma detachment in JET Ohmic discharges. Nuclear Materials and Energy, 2020, 25, 100836.	1.3	2
72	In-situ assessment of the emissivity of tungsten plasma facing components of the WEST tokamak. Nuclear Materials and Energy, 2020, 25, 100851.	1.3	14

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73	Boron transport simulation using the ERO2.0 code for real-time wall conditioning in the large helical device. Nuclear Materials and Energy, 2020, 25, 100853.	1.3	4
74	Erosion and screening of tungsten during inter/intra-ELM periods in the JET-ILW divertor. Nuclear Materials and Energy, 2020, 25, 100859.	1.3	7
75	Impurity sources and fluxes in W7-X: from the plasma-facing components to the edge layer. Physica Scripta, 2020, T171, 014040.	2.5	14
76	ERO2.0 modelling of the effects of surface roughness on molybdenum erosion and redeposition in the PSI-2 linear plasma device. Physica Scripta, 2020, T171, 014057.	2.5	19
77	Comparison of erosion and deposition in JET divertor during the first three ITER-like wall campaigns. Physica Scripta, 2020, T171, 014059.	2.5	19
78	Metallography and mechanical parameters of plasma-exposed plasma-facing materials and components. Physica Scripta, 2020, T171, 014042.	2.5	5
79	Inspection of W 7-X plasma-facing components after the operation phase OP1.2b: observations and first assessments. Physica Scripta, 2020, T171, 014033.	2.5	11
80	In-vessel colorimetry of Wendelstein 7-X first wall components: variation of layer deposition distribution in OP1.2a and OP1.2b. Physica Scripta, 2020, T171, 014054.	2.5	5
81	Scanning electron microscopy analyses of an ITER plasma-facing unit mockup exposed to extreme ion fluences in Magnum-PSI. Physica Scripta, 2020, T171, 014026.	2.5	8
82	Material erosion and deposition on the divertor of W7-X. Physica Scripta, 2020, T171, 014035.	2.5	20
83	ITER monoblock performance under lifetime loading conditions in Magnum-PSI. Physica Scripta, 2020, T171, 014065.	2.5	22
84	Peculiarity of highly radiating multi-impurity seeded H -mode plasmas on JET with ITER-like wall. Physica Scripta, 2020, T171, 014055.	2.5	10
85	Wall conditioning at the Wendelstein 7-X stellarator operating with a graphite divertor. Physica Scripta, 2020, T171, 014063.	2.5	15
86	Long pulse D_{2} and N_{2} seeded discharges on the upper actively cooled tungsten divertor of WEST. Physica Scripta, 2020, T171, 014074.	2.5	2
87	17th international conference on plasma-facing materials and components for fusion applications. Physica Scripta, 2020, T171, 010201.	2.5	0
88	First Monte Carlo modelling of global beryllium migration in ITER using ERO2.0. Contributions To Plasma Physics, 2020, 60, e201900149.	1.1	17
89	<i>Ex situ</i> analysis of W7-X divertor plasma-facing components by picosecond laser diagnostics. Physica Scripta, 2020, T171, 014018.	2.5	13
90	Increasing the density in Wendelstein 7-X: benefits and limitations. Nuclear Fusion, 2020, 60, 036020.	3.5	27

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91	Reversed-slit spectroscopy method for in situ measurement of H isotopes on plasma facing material. Journal of Instrumentation, 2020, 15, C01007-C01007.	1.2	0
92	Retarding field analyzer for the wendelstein 7-X boundary plasma. Fusion Engineering and Design, 2020, 157, 111623.	1.9	7
93	First efforts in numerical modeling of tungsten migration in WEST with SolEdge2D-EIRENE and ERO2.0. Physica Scripta, 2020, T171, 014013.	2.5	16
94	Optical isolation of spectral lines emitted by sputtered tungsten in a weakly magnetized plasma. Journal of Physics B: Atomic, Molecular and Optical Physics, 2020, 54, 025401.	1.5	5
95	Performance of Eurofer97 under deuterium plasma exposure with seeded impurities at elevated temperature. Physica Scripta, 2020, T171, 014071.	2.5	2
96	Infra-red thermography estimate of deposited heat load dynamics on the lower tungsten divertor of WEST. Physica Scripta, 2020, T171, 014046.	2.5	7
97	Efficiency of laser-induced desorption of D from Be/D layers and surface modifications due to LID. Physica Scripta, 2020, T171, 014075.	2.5	11
98	An in situ diagnostic method for monitoring of fuel retention on the first wall under long-pulse operation of experimental advanced superconducting tokamak. Physica Scripta, 2020, T171, 014069.	2.5	5
99	Tungsten divertor sources in WEST related to impurity inventory and local plasma conditions. Physica Scripta, 2020, T171, 014060.	2.5	12
100	Impact of boronizations on impurity sources and performance in Wendelstein 7-X. Nuclear Fusion, 2020, 60, 086007.	3.5	26
101	Effect of fuel isotope mass on q-profile formation in JET hybrid plasmas. Nuclear Fusion, 2020, 60, 086008.	3.5	12
102	Long discharges in a steady state with $D_{2₂}$ and $N_{2₂}$ on the actively cooled tungsten upper divertor in WEST. Nuclear Fusion, 2020, 60, 126046.	3.5	9
103	Interpretative transport modeling of the WEST boundary plasma: main plasma and light impurities. Nuclear Fusion, 2020, 60, 126048.	3.5	18
104	Light-reflection-induced changes in the line shape of sputtered atoms. Physica Scripta, 2020, T171, 014031.	2.5	3
105	Isotope effect on the detachment onset density in JET ohmic plasmas. Physica Scripta, 2020, T171, 014039.	2.5	1
106	Estimation of ELM effects on Be and W erosion at JET-ILW. Physica Scripta, 2020, T171, 014027.	2.5	1
107	Decommissioning of TEXTOR: properties of the Inconel liner. Physica Scripta, 2020, T171, 014036.	2.5	0
108	Deuterium retention in mixed Be-W-D codeposited layers. Nuclear Fusion, 2020, 60, 126005.	3.5	5

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109	Monitoring Removal of W Layer from Ag Substrate Using Balmer- $\text{H}\alpha$ Emission of Backscattered Hydrogen Atoms in Low Density Gas Discharge. Acta Physica Polonica A, 2020, 138, 643-649.	0.5	1
110	First divertor physics studies in Wendelstein 7-X. Nuclear Fusion, 2019, 59, 096014.	3.5	34
111	First demonstration of radiative power exhaust with impurity seeding in the island divertor at Wendelstein 7-X. Nuclear Fusion, 2019, 59, 106020.	3.5	23
112	Overview of physics studies on ASDEX Upgrade. Nuclear Fusion, 2019, 59, 112014.	3.5	38
113	Performance of Wendelstein 7-X stellarator plasmas during the first divertor operation phase. Physics of Plasmas, 2019, 26, .	1.9	83
114	Development and Performance of Tungsten-Coated Graphitic Foam for Plasma-Facing Components. Fusion Science and Technology, 2019, 75, 551-557.	1.1	3
115	Fuel Retention Diagnostic Setup (FREDIS) for desorption of gases from beryllium and tritium containing samples. Fusion Engineering and Design, 2019, 146, 1176-1180.	1.9	9
116	Overview of first Wendelstein 7-X high-performance operation. Nuclear Fusion, 2019, 59, 112004.	3.5	165
117	Erosion, screening, and migration of tungsten in the JET divertor. Nuclear Fusion, 2019, 59, 096035.	3.5	60
118	First Observation of a Stable Highly Dissipative Divertor Plasma Regime on the Wendelstein 7-X Stellarator. Physical Review Letters, 2019, 123, 025002.	7.8	33
119	Measurement of the edge ion temperature in W7-X with island divertor by a retarding field analyzer probe. Nuclear Fusion, 2019, 59, 126002.	3.5	11
120	Emission of Fast Hydrogen Atoms in a Low Density Gas Discharge – The Most “Natural” Mirror Laboratory. Atoms, 2019, 7, 81.	1.6	1
121	Quantitative analysis of elemental depth on Wendelstein 7-X divertor baffle screws by picosecond laser-induced breakdown spectroscopy. Spectrochimica Acta, Part B: Atomic Spectroscopy, 2019, 160, 105689.	2.9	17
122	Modelling of tungsten erosion and deposition in the divertor of JET-ILW in comparison to experimental findings. Nuclear Materials and Energy, 2019, 18, 239-244.	1.3	24
123	A locked mode indicator for disruption prediction on JET and ASDEX upgrade. Fusion Engineering and Design, 2019, 138, 254-266.	1.9	8
124	The software and hardware architecture of the real-time protection of in-vessel components in JET-ILW. Nuclear Fusion, 2019, 59, 076016.	3.5	9
125	Dependence on plasma shape and plasma fueling for small edge-localized mode regimes in TCV and ASDEX Upgrade. Nuclear Fusion, 2019, 59, 086020.	3.5	34
126	Beryllium global erosion and deposition at JET-ILW simulated with ERO2.0. Nuclear Materials and Energy, 2019, 18, 331-338.	1.3	36

127	Operation of probe heads on the Multi-Purpose-Manipulator at W7-X in OP 1.2a. Fusion Engineering and Design, 2019, 146, 2353-2355.	1.9	4
128	Modelling of the effect of ELMs on fuel retention at the bulk W divertor of JET. Nuclear Materials and Energy, 2019, 19, 397-402.	1.3	7
129	Spectroscopic studies of fuel recycling and impurity behaviors in the divertor region of Wendelstein 7-X. Plasma Science and Technology, 2019, 21, 105102.	1.5	3
130	Overview of the JET preparation for deuterium-tritium operation with the ITER like-wall. Nuclear Fusion, 2019, 59, 112021.	3.5	87
131	Edge plasma measurements on the OP 1.2a divertor plasmas at W7-X using the combined probe. Nuclear Materials and Energy, 2019, 19, 179-183.	1.3	15
132	EDGE2D-EIRENE predictions of molecular emission in DIII-D high-recycling divertor plasmas. Nuclear Materials and Energy, 2019, 19, 211-217.	1.3	19
133	Development of glow discharge and electron cyclotron resonance heating conditioning on W7-X. Nuclear Materials and Energy, 2019, 18, 227-232.	1.3	8
134	An assessment of nitrogen concentrations from spectroscopic measurements in the JET and ASDEX upgrade divertor. Nuclear Materials and Energy, 2019, 18, 147-152.	1.3	8
135	Investigation of 3D effects on heat fluxes in performance-optimized island divertor configurations at Wendelstein 7-X. Nuclear Materials and Energy, 2019, 18, 262-267.	1.3	21
136	Improved ERO modelling of beryllium erosion at ITER upper first wall panel using JET-ILW and PISCES-B experience. Nuclear Materials and Energy, 2019, 19, 510-515.	1.3	15
137	COREDIV numerical simulation of high neutron rate JET-ILW DD pulses in view of extension to JET-ILW DT experiments. Nuclear Fusion, 2019, 59, 056026.	3.5	4
138	Physics affecting heavy impurity migration in tokamaks: Benchmarking test-ion code ASCOT against TEXTOR tracer experiment. Nuclear Materials and Energy, 2019, 19, 307-315.	1.3	1
139	Laser-Induced Desorption of co-deposited Deuterium in Beryllium Layers on Tungsten. Nuclear Materials and Energy, 2019, 19, 503-509.	1.3	15
140	Surface roughness effect on Mo physical sputtering and re-deposition in the linear plasma device PSI-2 predicted by ERO2.0. Nuclear Materials and Energy, 2019, 19, 13-18.	1.3	27
141	Chemically assisted physical sputtering of tungsten: identification via the Si^+ ion signal	1.3	8
142	Tritium distributions on W-coated divertor tiles used in the third JET ITER-like wall campaign. Nuclear Materials and Energy, 2019, 18, 258-261.	1.3	10
143	Population modelling of the He II energy levels in tokamak plasmas: I. Collisional excitation model. Journal of Physics B: Atomic, Molecular and Optical Physics, 2019, 52, 045001.	1.5	1

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145	Micro-structured tungsten: an advanced plasma-facing material. Nuclear Materials and Energy, 2019, 19, 7-12.	1.3	16
146	Erosion and deposition investigations on Wendelstein 7-X first wall components for the first operation phase in divertor configuration. Fusion Engineering and Design, 2019, 146, 242-245.	1.9	17
147	Impact of ICRF on the scrape-off layer and on plasma wall interactions: From present experiments to fusion reactor. Nuclear Materials and Energy, 2019, 18, 131-140.	1.3	34
148	An upgraded LIBS system on linear plasma device PSI-2 for in situ diagnostics of plasma-facing materials. Fusion Engineering and Design, 2019, 146, 96-99.	1.9	9
149	Analysis of the outer divertor hot spot activity in the protection video camera recordings at JET. Fusion Engineering and Design, 2019, 139, 115-123.	1.9	3
150	Material migration and fuel retention studies during the JET carbon divertor campaigns. Fusion Engineering and Design, 2019, 138, 78-108.	1.9	25
151	Diagnostic setup for the divertor manipulator at Wendelstein 7-X. Nuclear Materials and Energy, 2019, 18, 77-81.	1.3	7
152	Determination of tungsten sources in the JET-ILW divertor by spectroscopic imaging in the presence of a strong plasma continuum. Nuclear Materials and Energy, 2019, 18, 118-124.	1.3	16
153	Depth resolved analysis of hydrogen in W7-X graphite components using laser-induced ablation-quadrupole mass spectrometry (LIA-QMS). Nuclear Materials and Energy, 2019, 18, 153-158.	1.3	14
154	Improved neutron activation dosimetry for fusion. Fusion Engineering and Design, 2019, 139, 109-114.	1.9	7
155	First results from divertor operation in Wendelstein 7-X. Plasma Physics and Controlled Fusion, 2019, 61, 014035.	2.1	75
156	Wall conditioning by ECRH discharges and He-GDC in the limiter phase of Wendelstein 7-X. Nuclear Fusion, 2018, 58, 066013.	3.5	15
157	Probe manipulators for Wendelstein 7-X and their interaction with the magnetic topology. Plasma Science and Technology, 2018, 20, 054002.	1.5	0
158	Ablation mass features in multi-pulses femtosecond laser ablate molybdenum target. Nuclear Instruments & Methods in Physics Research B, 2018, 418, 54-59.	1.4	11
159	Neutron spectroscopy measurements of 14 MeV neutrons at unprecedented energy resolution and implications for deuterium-tritium fusion plasma diagnostics. Measurement Science and Technology, 2018, 29, 045502.	2.6	35
160	14 MeV calibration of JET neutron detectors—phase 1: calibration and characterization of the neutron source. Nuclear Fusion, 2018, 58, 026012.	3.5	22
161	Emission of fast hydrogen atoms at a plasma–solid interface in a low density plasma containing noble gases. Journal of Physics B: Atomic, Molecular and Optical Physics, 2018, 51, 025702.	1.5	10
162	Depth-resolved sample composition analysis using laser-induced ablation-quadrupole mass spectrometry and laser-induced breakdown spectroscopy. Spectrochimica Acta, Part B: Atomic Spectroscopy, 2018, 144, 38-45.	2.9	14

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163	Determination of volumetric plasma parameters from spectroscopic N II and N III line ratio measurements in the ASDEX Upgrade divertor. Nuclear Fusion, 2018, 58, 016047.	3.5	14
164	Modelling of plasma-wall interaction and impurity transport in fusion devices and prompt deposition of tungsten as application. Plasma Physics and Controlled Fusion, 2018, 60, 014041.	2.1	31
165	Wall conditioning throughout the first carbon divertor campaign on Wendelstein 7-X. Nuclear Materials and Energy, 2018, 17, 235-241.	1.3	14
166	Impact of Kr and Ar seeding on D retention in ferritic-martensitic steels after high-fluence plasma exposure. Nuclear Materials and Energy, 2018, 17, 307-313.	1.3	2
167	On the role of finite grid extent in SOLPS-ITER edge plasma simulations for JET H-mode discharges with metallic wall. Nuclear Materials and Energy, 2018, 17, 174-181.	1.3	8
168	Plasma exposures of a high-conductivity graphitic foam for plasma facing components. Nuclear Materials and Energy, 2018, 17, 123-128.	1.3	4
169	The influence of helium on deuterium retention in beryllium co-deposits. Journal of Nuclear Materials, 2018, 512, 25-30.	2.7	10
170	Review on global migration, fuel retention and modelling after TEXTOR decommission. Nuclear Materials and Energy, 2018, 17, 83-112.	1.3	9
171	Synthetic spectra of BeH, BeD and BeT for emission modeling in JET plasmas. Journal of Physics B: Atomic, Molecular and Optical Physics, 2018, 51, 185701.	1.5	17
172	Magnetic configuration effects on the Wendelstein 7-X stellarator. Nature Physics, 2018, 14, 855-860.	16.7	110
173	An ultraviolet-visible-near infrared overview spectroscopy for divertor plasma diagnosis on Wendelstein 7-X. AIP Advances, 2018, 8, 085011.	1.3	5
174	Laser induced ablation spectroscopy for in situ characterization of the first wall on EAST tokamak. Fusion Engineering and Design, 2018, 135, 95-101.	1.9	18
175	High power neon seeded JET discharges: Experiments and simulations. Nuclear Materials and Energy, 2017, 12, 882-886.	1.3	13
176	Modelling of deposition and erosion of injected WF6 and MoF6 in TEXTOR. Nuclear Materials and Energy, 2017, 12, 564-568.	1.3	4
177	Surface modification of He pre-exposed tungsten samples by He plasma impact in the divertor manipulator of ASDEX Upgrade. Nuclear Materials and Energy, 2017, 12, 575-581.	1.3	18
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