

Boris Z Margolin

List of Publications by Year in Descending Order

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The third column is the impact factor (IF) of the journal, and the fourth column is the number of citations of the article.

100
papers

631
citations

15
h-index

21
g-index

104
ext. papers

657
ext. citations

1.3
avg, IF

3.59
L-index

#	Paper	IF	Citations
100	On the link of the embrittlement mechanisms and microcrack nucleation and propagation properties for RPV steels. Part I. Materials, study strategy and deformation properties. <i>Engineering Fracture Mechanics</i> , 2022 , 108400	4.2	
99	On the link of the embrittlement mechanisms and microcrack nucleation and propagation properties for RPV steels. Part II. Fracture properties and modelling. <i>Engineering Fracture Mechanics</i> , 2022 , 270, 108556	4.2	
98	Mechanisms of plastic deformation and fracture of austenitic chromium-nickel steel irradiated during 45 years in WWER-440. <i>Journal of Nuclear Materials</i> , 2021 , 549, 152911	3.3	5
97	On the Modelling of Thermal Aging through Neutron Irradiation and Annealing. <i>Advances in Materials Science and Engineering</i> , 2018 , 2018, 1-9	1.5	5
96	Physical and mechanical modeling and prediction of fracture strain and fracture toughness of irradiated austenitic steels. <i>Engineering Failure Analysis</i> , 2015 , 47, 283-298	3.2	2
95	Physical and mechanical modelling of neutron irradiation effect on ductile fracture. Part 1. Prediction of fracture strain and fracture toughness of austenitic steels. <i>Journal of Nuclear Materials</i> , 2014 , 452, 595-606	3.3	15
94	Effect of neutron irradiation on tensile properties of materials for pressure vessel internals of WWER type reactors. <i>Journal of Nuclear Materials</i> , 2014 , 444, 373-384	3.3	23
93	On the nature of drastic strength reduction of austenitic steels during irradiation-induced swelling. <i>Strength of Materials</i> , 2013 , 45, 257-270	0.6	
92	A Physical-Mechanical Model of Ductile Fracture in Irradiated Austenitic Steels. <i>Strength of Materials</i> , 2013 , 45, 125-143	0.6	8
91	Analysis of a link of embrittlement mechanisms and neutron flux effect as applied to reactor pressure vessel materials of WWER. <i>Journal of Nuclear Materials</i> , 2013 , 434, 347-356	3.3	22
90	Approaches to Substantiated Service Life Extension for BN Reactors. <i>Strength of Materials</i> , 2013 , 45, 442-447	0.6	
89	Analysis of Relationship Between the Radiation Embrittlement Mechanisms and the Influence of Neutron Flux in Respect of WWER Reactor Pressure Vessel Materials. <i>Strength of Materials</i> , 2013 , 45, 406-423	0.6	2
88	Radiation embrittlement modelling in multi-scale approach to brittle fracture of RPV steels. <i>International Journal of Fracture</i> , 2013 , 179, 87-108	2.3	38
87	Fracture toughness prediction for highly irradiated RPV materials: From test results to RPV integrity assessment. <i>Journal of Nuclear Materials</i> , 2013 , 432, 313-322	3.3	8
86	Analysis of embrittlement of WWER-1000 RPV materials. <i>International Journal of Pressure Vessels and Piping</i> , 2012 , 89, 178-186	2.4	30
85	TAREG 2.01/00 project, Validation of neutron embrittlement for VVER 1000 and 440/213 RPVs, with emphasis on integrity assessment. <i>Progress in Nuclear Energy</i> , 2012 , 58, 52-57	2.3	7
84	Analysis of the influence of type of stress state on radiation swelling and radiation creep of austenitic steels. <i>Strength of Materials</i> , 2012 , 44, 227-240	0.6	21

83	A study of crack propagation in austenitic steels under creep conditions including the influence of thermal pre-ageing. <i>Strength of Materials</i> , 2012 , 44, 585-599	0.6	
82	Special features of calculation of C*-integral in thermomechanical loading of structural elements. <i>Strength of Materials</i> , 2012 , 44, 347-358	0.6	
81	A method of strength assessment of WWER reactor internals by the criterion of stress corrosion cracking in irradiated austenitic steels. <i>Strength of Materials</i> , 2012 , 44, 115-128	0.6	11
80	A comparative analysis of radiation-thermal forming for reflectors of reactors BN-600 and BN-800 by results of numerical simulation. <i>Journal of Machinery Manufacture and Reliability</i> , 2011 , 40, 585-591	0.6	1
79	A new approach to description of in-service embrittlement of WWER-1000 reactor pressure vessel materials. <i>Strength of Materials</i> , 2010 , 42, 2-16	0.6	6
78	Embrittlement and fracture toughness of highly irradiated austenitic steels for vessel internals of WWER type reactors. Part 2. Relation between irradiation swelling and irradiation embrittlement. Physical and mechanical behavior. <i>Strength of Materials</i> , 2010 , 42, 144-153	0.6	25
77	Embrittlement and fracture toughness of highly irradiated austenitic steels for vessel internals of WWER type reactors. Part 3. Analysis of crack propagation conditions. <i>Strength of Materials</i> , 2010 , 42, 258-271	0.6	2
76	Brittle fracture local criterion and radiation embrittlement of reactor pressure vessel steels. <i>Strength of Materials</i> , 2010 , 42, 506-527	0.6	4
75	Analysis of applicability of small-sized specimens to prediction of temperature dependence of fracture toughness. <i>Strength of Materials</i> , 2009 , 41, 119-134	0.6	6
74	A study of suitability of various criteria for fracture toughness prediction on small-sized specimens. <i>Strength of Materials</i> , 2009 , 41, 345-355	0.6	4
73	Embrittlement and fracture toughness of highly irradiated austenitic steels for vessel internals of WWER type reactors. Part 1. Relation between irradiation swelling and irradiation embrittlement. Experimental results. <i>Strength of Materials</i> , 2009 , 41, 593-602	0.6	16
72	Prediction of Creep-Rupture Properties for Austenitic Steels Undergone Neutron Irradiation 2009 ,		1
71	Structural Integrity Assessment of WWER Internals on Stress Corrosion Cracking Criterion 2009 ,		2
70	The Relationship of Radiation Embrittlement and Swelling for Austenitic Steels for WWER Internals 2009 ,		4
69	Modification of Pre-Cracked Charpy Specimens for Surveillance Specimen Programs 2009 ,		2
68	On some criterial problems of fatigue crack initiation and growth in polycrystals. <i>Strength of Materials</i> , 2008 , 40, 397-410	0.6	1
67	A method for predicting fracture resistance of material in cyclic loading under viscoelastoplastic deformation and neutron irradiation conditions. <i>Strength of Materials</i> , 2008 , 40, 601-614	0.6	7
66	Prometey local approach to brittle fracture: Development and application. <i>Engineering Fracture Mechanics</i> , 2008 , 75, 3483-3498	4.2	43

65	Physical and Mechanical Aspects of Radiation Embrittlement of RPV Steels 2008 ,		4
64	Development of Prometey local approach and analysis of physical and mechanical aspects of brittle fracture of RPV steels. <i>International Journal of Pressure Vessels and Piping</i> , 2007 , 84, 320-336	2.4	16
63	Application of a new cleavage fracture criterion for fracture toughness prediction for RPV steels. <i>Fatigue and Fracture of Engineering Materials and Structures</i> , 2006 , 29, 697-713	3	21
62	Modeling for fracture in materials under long-term static creep loading and neutron irradiation. Part 1. A physico-mechanical model. <i>Strength of Materials</i> , 2006 , 38, 221-233	0.6	11
61	Modeling for fracture in materials under long-term static creep loading and neutron irradiation. Part 2. Prediction of creep rupture strength for austenitic materials. <i>Strength of Materials</i> , 2006 , 38, 449-457	0.6	11
60	Modeling for fracture in materials under long-term static creep loading and neutron irradiation. Part 3. Crack growth rate prediction for austenitic materials. <i>Strength of Materials</i> , 2006 , 38, 565-574	0.6	8
59	Prediction of the dependence of KJC(T) on neutron fluence for RPV steels on the basis of the Unified Curve concept. <i>International Journal of Pressure Vessels and Piping</i> , 2005 , 82, 679-686	2.4	11
58	Probabilistic definition of local criterion for brittle fracture under complex thermomechanical loading. <i>Strength of Materials</i> , 2005 , 37, 16-29	0.6	
57	Prediction of Temperature Dependence of Fracture Toughness as a Function of Neutron Fluence for Pressure-Vessel Steels by Using the Unified Curve Method. <i>Strength of Materials</i> , 2005 , 37, 243-253	0.6	
56	Analysis of structure integrity of RPV on the basis of brittle fracture criterion: new approaches. <i>International Journal of Pressure Vessels and Piping</i> , 2004 , 81, 651-656	2.4	11
55	Prediction of brittle fracture of RPV steels under complex loading on the basis of a local probabilistic approach. <i>International Journal of Pressure Vessels and Piping</i> , 2004 , 81, 949-959	2.4	9
54	Investigation of nanostructure of reactor pressure vessel steel with different degree of embrittlement. <i>Physica B: Condensed Matter</i> , 2004 , 350, E471-E474	2.8	1
53	Neutron Embrittlement of VVER 1000 and 440/213 RPVs: Learning From EC Projects on RPV Integrity 2004 , 77		
52	Investigation of Residual Stresses Caused by Welding, Cladding and Tempering of Reactor Pressure Vessels 2003 , 3		
51	A New Engineering Method for Prediction of Fracture Toughness Temperature Dependence for Pressure-Vessel Steels. <i>Strength of Materials</i> , 2003 , 35, 440-457	0.6	6
50	Temperature Dependence of Brittle Fracture Toughness of Reactor Pressure-Vessel Steels upon Ductile Crack Growth. <i>Strength of Materials</i> , 2003 , 35, 14-23	0.6	1
49	A new engineering method for prediction of the fracture toughness temperature dependence for RPV steels. <i>International Journal of Pressure Vessels and Piping</i> , 2003 , 80, 817-829	2.4	34
48	The effect of ductile crack growth on the temperature dependence of cleavage fracture toughness for a RPV steel with various degrees of embrittlement. <i>International Journal of Pressure Vessels and Piping</i> , 2003 , 80, 285-296	2.4	13

47	Local Approach of Fracture in the Ductile Regime and Application to VVER Materials 2002 , 413		
46	Fracture toughness predictions for a reactor pressure vessel steel in the initial and highly embrittled states with the Master Curve approach and a probabilistic model. <i>International Journal of Pressure Vessels and Piping</i> , 2002 , 79, 219-231	2.4	23
45	Prediction of Fracture Toughness of Reactor Pressure-Vessel Steels Using the Master Curve Concept and Probabilistic Model. <i>Strength of Materials</i> , 2002 , 34, 1-11	0.6	4
44	Modeling of Ductile Crack Growth in Reactor Pressure-Vessel Steels and Determination of JR Curves. <i>Strength of Materials</i> , 2002 , 34, 120-130	0.6	2
43	Application of Local Approach Concept of Cleavage Fracture to VVER Materials 2002 , 113		8
42	Cleavage fracture toughness for 3Cr15MoV reactor pressure vessel steel: theoretical prediction and experimental investigation. <i>International Journal of Pressure Vessels and Piping</i> , 2001 , 78, 429-441	2.4	5
41	Simulation of JR-curves for reactor pressure vessels steels on the basis of a ductile fracture model. <i>International Journal of Pressure Vessels and Piping</i> , 2001 , 78, 715-725	2.4	14
40	Prediction of the Brittle Fracture Toughness of Neutron-Irradiated Reactor Pressure Vessel Steels. Part 2. <i>Strength of Materials</i> , 2001 , 33, 201-206	0.6	
39	Prediction of the Brittle Fracture Toughness of Neutron-Irradiated Reactor Pressure-Vessel Steels. Part 1. <i>Strength of Materials</i> , 2001 , 33, 95-105	0.6	
38	Prediction of Ductile Fracture Toughness for Neutron-Irradiated Reactor Pressure-Vessel Steels. Part 1. <i>Strength of Materials</i> , 2001 , 33, 318-324	0.6	
37	Prediction of Ductile Fracture Toughness for Neutron-Irradiated Reactor Pressure-Vessel Steels. Part 2. <i>Strength of Materials</i> , 2001 , 33, 407-415	0.6	
36	Determination of residual stress and strain fields caused by cladding and tempering of reactor pressure vessels. <i>International Journal of Pressure Vessels and Piping</i> , 2000 , 77, 723-735	2.4	22
35	Modeling for ductile-to-brittle transition under ductile crack growth for reactor pressure vessel steels. <i>International Journal of Pressure Vessels and Piping</i> , 1999 , 76, 309-317	2.4	4
34	Radiation embrittlement modelling for reactor pressure vessel steels: I. Brittle fracture toughness prediction. <i>International Journal of Pressure Vessels and Piping</i> , 1999 , 76, 715-729	2.4	25
33	Radiation embrittlement modelling for reactor pressure vessel steels: II. Ductile fracture toughness prediction. <i>International Journal of Pressure Vessels and Piping</i> , 1999 , 76, 731-740	2.4	5
32	Lifetime prediction for intercrystalline fracture under cyclic loading with various strain rates. <i>International Journal of Fatigue</i> , 1999 , 21, 497-505	5	4
31	Probabilistic prediction of the crack resistance of nuclear pressure vessel steels on the basis of a local approach. Part 1. <i>Strength of Materials</i> , 1999 , 31, 1-12	0.6	1
30	Prediction of ductile-brittle transition for ductile crack growth in reactor pressure-vessel steels. <i>Strength of Materials</i> , 1999 , 31, 525-538	0.6	

29	Probabilistic prediction of the crack resistance of nuclear pressure-vessel steels on the basis of a local approach. Part 2. <i>Strength of Materials</i> , 1999 , 31, 107-119	0.6	3
28	Analysis of the effect of biaxial loading on the fracture toughness of reactor pressure-vessel steels. <i>Strength of Materials</i> , 1999 , 31, 433-447	0.6	2
27	Analysis of biaxial loading effect on fracture toughness of reactor pressure vessel steels. <i>International Journal of Pressure Vessels and Piping</i> , 1998 , 75, 589-601	2.4	15
26	The use of the T^* -integral to simulate subcritical crack growth taking into account the evolution of pores in the material. <i>Strength of Materials</i> , 1997 , 29, 209-219	0.6	
25	Preliminary compression of a material as a factor in changing the brittle fracture mechanism for BCC metals. <i>Strength of Materials</i> , 1996 , 28, 251-261	0.6	4
24	Stress-strain curves of polycrystals: Analysis of hardening and softening. <i>Strength of Materials</i> , 1995 , 27, 580-591	0.6	
23	Analysis of certain problems of the brittle fracture of bcc metals. <i>Strength of Materials</i> , 1994 , 26, 477-491	0.6	
22	Features of the deformation and failure of welded joints with pulsed loading. <i>Strength of Materials</i> , 1993 , 25, 330-333	0.6	
21	Creep and failure of structurally stable materials with nonstationary loading and all-round compression. <i>Strength of Materials</i> , 1993 , 25, 86-95	0.6	1
20	Computational analysis of crack development during ductile failure. <i>Strength of Materials</i> , 1992 , 24, 577-586	0.6	
19	Simulation of the condition of plane sections in the finite element method. <i>Strength of Materials</i> , 1992 , 24, 362-365	0.6	
18	Brittle fracture criterion: Structural mechanics approach. <i>Strength of Materials</i> , 1992 , 24, 115-131	0.6	2
17	Effect of cyclic deformation on the resistance of a material to brittle failure. <i>Strength of Materials</i> , 1991 , 23, 14-23	0.6	
16	Effect of strain rate on the nature of failure in static and cyclic loading. Report 1. Formulation of general approaches. <i>Strength of Materials</i> , 1991 , 23, 107-121	0.6	2
15	Influence of strain rate on the nature of failure under prolonged static and cyclical loads. 2. Examples of calculation. <i>Strength of Materials</i> , 1991 , 23, 876-883	0.6	1
14	Physicomechanical model of creep-induced fracture. <i>Strength of Materials</i> , 1990 , 22, 1409-1418	0.6	
13	Solving a dynamic elastoplastic problem of fracture mechanics by the finite element method. 1. The dynamic elastoplastic problem. <i>Strength of Materials</i> , 1990 , 22, 935-943	0.6	1
12	Propagation of fatigue cracks in mixed loading. <i>Strength of Materials</i> , 1990 , 22, 309-316	0.6	

11	Analysis of the origin and development of fatigue failure in pearlitic steels. <i>Strength of Materials</i> , 1990 , 22, 478-490	0.6	1
10	Several physicommechanical approaches to the analysis of macroscopic failure. 2. Ductile failure. <i>Strength of Materials</i> , 1989 , 21, 965-974	0.6	
9	An analysis of the conditions of brittle fracture origin. <i>Strength of Materials</i> , 1989 , 21, 1439-1445	0.6	
8	Physicommechanical approaches to an analysis of macroscopic failure criteria. Report 3. Brittle failure. <i>Strength of Materials</i> , 1989 , 21, 841-852	0.6	1
7	Physical-mechanical approaches to the analysis of macroscopic failure. Report 1. Fatigue failure. <i>Strength of Materials</i> , 1989 , 21, 703-712	0.6	
6	Analysis of special features of deformation of the material at the crack tip and of criteria of fatigue fracture propagation with an allowance made for structural parameters. Report 1. <i>Strength of Materials</i> , 1988 , 20, 992-1000	0.6	
5	Analysis of special features of deformation of the material at the crack tip and of criteria of fatigue fracture propagation with an allowance made for structural parameters. Report 2. <i>Strength of Materials</i> , 1988 , 20, 1001-1009	0.6	
4	A mechanical model of fatigue crack propagation. Report 1. <i>Strength of Materials</i> , 1985 , 17, 1037-1043	0.6	
3	A mechanical model of fatigue crack propagation. Report 2. <i>Strength of Materials</i> , 1985 , 17, 1044-1049	0.6	
2	Propagation of fatigue cracks in tee welded joints taking into account welding stresses. <i>Strength of Materials</i> , 1983 , 15, 1596-1600	0.6	
1	Determination of the crack path and intensity of elastic energy released during cyclic loading taking account of welding stresses. <i>Strength of Materials</i> , 1983 , 15, 1322-1328	0.6	2