

Lionel Desgranges

List of Publications by Year in descending order

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108
papers

2,302
citations

218381

26
h-index

233125

45
g-index

111
all docs

111
docs citations

111
times ranked

1575
citing authors

#	ARTICLE	IF	CITATIONS
1	Annealing of the Raman defect peaks in He-implanted UO ₂ . Journal of Nuclear Materials, 2022, 559, 153405.	1.3	7
2	MERARG Experimental Loop: A Forward Fitting Method for Fission Gas Release Analysis. IEEE Transactions on Nuclear Science, 2022, 69, 786-791.	1.2	0
3	In-situ high resolution photoelectron spectroscopy study on interaction of sodium with UO _{2+x} film (0 ≤ x ≤ 1). Journal of Nuclear Materials, 2021, 545, 152646.	1.3	8
4	Atomic scale Monte-Carlo simulations of neutron diffraction experiments on stoichiometric uranium dioxide up to 1664 K. Nuclear Instruments and Methods in Physics Research, Section A: Accelerators, Spectrometers, Detectors and Associated Equipment, 2021, 1002, 165251.	0.7	3
5	Gas detection in sodium cooled fast reactors: determination of a transfer function. EPJ Web of Conferences, 2021, 253, 05002.	0.1	1
6	On the origins and the evolution of the fuel-cladding bonding phenomenon in PWR fuel rods. Journal of Nuclear Materials, 2019, 520, 110-120.	1.3	11
7	Characterization of an UO ₂ ceramic via Raman imaging and electron back-scattering diffraction. Materials Characterization, 2019, 147, 280-285.	1.9	12
8	Quantification of irradiation-induced defects in UO ₂ using Raman and positron annihilation spectroscopies. Acta Materialia, 2019, 164, 512-519.	3.8	23
9	Thermochemical effect of fission products on sodium ²³ Na MOX fuel reaction: The case of niobium. Journal of Nuclear Materials, 2018, 500, 361-365.	1.3	0
10	Understanding Local Structure versus Long-Range Structure: The Case of UO ₂ . Chemistry - A European Journal, 2018, 24, 2085-2088.	1.7	3
11	Evaluation of T _{2g} band intensity distribution across a surface of an UO ₂ ceramic. AIP Conference Proceedings, 2018, , .	0.3	1
12	New insight in the uranium valence state determination in U _{1-x} Nd _x O _{2±} . Journal of Nuclear Materials, 2018, 507, 145-150.	1.3	18
13	Investigating the role of irradiation defects during UO ₂ oxidative dissolution. Journal of Nuclear Materials, 2018, 509, 305-312.	1.3	6
14	A new methodology for studying neutron absorber materials: First results with boron carbide. Nuclear Instruments & Methods in Physics Research B, 2018, 432, 42-47.	0.6	5
15	Characterization of Oxygen Defect Clusters in UO _{2+x} Using Neutron Scattering and PDF Analysis. Inorganic Chemistry, 2018, 57, 7064-7076.	1.9	11
16	Raman imaging and principal component analysis-based data processing on uranium oxide ceramics. Materials Characterization, 2017, 129, 260-269.	1.9	40
17	What Is the Actual Local Crystalline Structure of Uranium Dioxide, UO ₂ ? A New Perspective for the Most Used Nuclear Fuel. Inorganic Chemistry, 2017, 56, 321-326.	1.9	45
18	Fission products behaviour during a power transient: Their inventory in an intragranular bubble. Journal of Nuclear Materials, 2017, 493, 225-229.	1.3	3

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19	High Burn-up Structure in Nuclear Fuel: Impact on Fuel Behavior. EPJ Web of Conferences, 2016, 115, 04005.	0.1	5
20	Thin film samples: a new methodology for investigating the mechanisms of fission gas releases from nuclear fuel during a LOCA. MRS Advances, 2016, 1, 2439-2445.	0.5	0
21	Irradiation Defects in UO ₂ Leached in Oxidizing Water: An in-situ Raman Study. Procedia Chemistry, 2016, 21, 326-333.	0.7	4
22	Storage of Defective Fuel Pins in SFR Core. Procedia Chemistry, 2016, 21, 378-385.	0.7	3
23	Experimental evidence of oxygen thermo-migration in PWR UO ₂ fuels during power ramps using in-situ oxido-reduction indicators. Journal of Nuclear Materials, 2016, 480, 32-39.	1.3	10
24	In situ Raman estimation of irradiation-induced heating of UO ₂ . Journal of Nuclear Materials, 2016, 478, 172-175.	1.3	3
25	Structural Changes in the Local Environment of Uranium Atoms in the Three Phases of U ₄ O ₉ . Inorganic Chemistry, 2016, 55, 7485-7491.	1.9	19
26	Charged defects during alpha-irradiation of actinide oxides as revealed by Raman and luminescence spectroscopy. Nuclear Instruments & Methods in Physics Research B, 2016, 374, 67-70.	0.6	26
27	Evolution of extended defects in polycrystalline UO ₂ under heavy ion irradiation: combined TEM, XRD and Raman study. Nuclear Instruments & Methods in Physics Research B, 2016, 374, 51-57.	0.6	32
28	High temperature Raman study of UO ₂ : A possible tool for <i>in situ</i> estimation of irradiation-induced heating. Journal of Raman Spectroscopy, 2015, 46, 418-420.	1.2	24
29	Characterization of radiation damage in ceramics: Old challenge new issues?. Journal of Materials Research, 2015, 30, 1495-1515.	1.2	21
30	Raman micro-spectroscopy of UOX and MOX spent nuclear fuel characterization and oxidation resistance of the high burn-up structure. Journal of Nuclear Materials, 2015, 458, 343-349.	1.3	28
31	Characterising the U-Nd-O miscibility gap by an experimental and a theoretical approach. Journal of Nuclear Materials, 2015, 458, 394-405.	1.3	7
32	Evidence of a new crystalline phase in U-Gd-O phase diagram. Journal of Nuclear Materials, 2015, 461, 186-192.	1.3	2
33	Experimental evidence of the formation of a new chemical phase in a power ramped UO ₂ nuclear fuel. Journal of Nuclear Materials, 2015, 457, 246-251.	1.3	3
34	3D thermo-chemical-mechanical simulation of power ramps with ALCYONE fuel code. Journal of Nuclear Materials, 2014, 452, 578-594.	1.3	41
35	What Can We Learn From Raman Spectroscopy on Irradiation-Induced Defects in UO ₂ ?. Jom, 2014, 66, 2546-2552.	0.9	13
36	Application of the CALPHAD method to the thermodynamic modeling of a miscibility gap in the U-Nd-O phase diagram. Materials Research Society Symposia Proceedings, 2014, 1645, 1.	0.1	1

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37	A statistical approach of the thermodynamic properties of UO ₂ at high temperature. Nuclear Instruments & Methods in Physics Research B, 2014, 327, 68-73.	0.6	4
38	Characterization of Nuclear Materials in Extreme Conditions: Raman Spectroscopy Approach. IEEE Transactions on Nuclear Science, 2014, 61, 2045-2051.	1.2	19
39	Existence of a miscibility gap in uranium neodymium oxide materials used as nuclear fuels simulants. Progress in Nuclear Energy, 2014, 72, 22-26.	1.3	5
40	Annealing of the defects observed by Raman spectroscopy in UO ₂ irradiated by 25MeV He ²⁺ ions. Nuclear Instruments & Methods in Physics Research B, 2014, 327, 74-77.	0.6	22
41	Measurement of energy spectra on irradiated polycrystalline UO ₂ samples using secondary ion mass spectrometry. Surface and Interface Analysis, 2013, 45, 427-429.	0.8	0
42	Changes in uranium energy spectra measured by SIMS on slightly hyperstoichiometric UO ₂ . Surface and Interface Analysis, 2013, 45, 420-422.	0.8	0
43	A possible new mechanism for defect formation in irradiated UO ₂ . Nuclear Instruments & Methods in Physics Research B, 2013, 315, 169-172.	0.6	27
44	Evidence of tellurium iodide compounds in a power-ramped irradiated UO ₂ fuel rod. Journal of Nuclear Materials, 2013, 437, 409-414.	1.3	14
45	Density Functional Theory Calculations of UO ₂ Oxidation: Evolution of UO _{2+x} , U ₄ O ₉ , U ₃ O ₇ , and U ₃ O ₈ . Inorganic Chemistry, 2013, 52, 2769-2778.	1.9	96
46	Characterization of nuclear materials in extreme conditions: The raman spectroscopy approach. , 2013, , .		0
47	Reduction of Gd ₆ UO ₁₂ for the Synthesis of Gd ₆ UO ₁₁ . Materials Research Society Symposia Proceedings, 2013, 1514, 151-156.	0.1	0
48	In situ Raman monitoring of He ²⁺ irradiation induced damage in a UO ₂ ceramic. Applied Physics Letters, 2013, 103, 041904.	1.5	33
49	Structural features in fluorite compounds relevant for nuclear applications. Materials Research Society Symposia Proceedings, 2012, 1383, 35.	0.1	1
50	Determination of in-depth damaged profile by Raman line scan in a pre-cut He ²⁺ irradiated UO ₂ . Applied Physics Letters, 2012, 100, .	1.5	80
51	Miscibility Gap in the U-Nd-O Phase Diagram: a New Approach of Nuclear Oxides in the Environment?. Inorganic Chemistry, 2012, 51, 9147-9149.	1.9	44
52	Raman spectrum of U ₄ O ₉ : a new interpretation of damage lines in UO ₂ . Journal of Raman Spectroscopy, 2012, 43, 455-458.	1.2	127
53	Is UO ₂ irradiation resistance due to its unusual high temperature behaviour?. Journal of Nuclear Materials, 2012, 420, 334-337.	1.3	8
54	How polarons can enhance UO ₂ irradiation resistance?. Nuclear Instruments & Methods in Physics Research B, 2012, 277, 109-111.	0.6	5

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55	Refinement of the $\pm U_4O_9$ Crystalline Structure: New Insight into the $U_4O_9 \rightarrow U_3O_8$ Transformation. Inorganic Chemistry, 2011, 50, 6146-6151.	1.9	52
56	First experimental evidence by SIMS of different surface binding energies for uranium according to its oxidation state. Applied Surface Science, 2011, 257, 6208-6211.	3.1	7
57	Raman spectroscopy characterization of actinide oxides ($U^{IV}Pu^{III}O_2$): Resistance to oxidation by the laser beam and examination of defects. Journal of Nuclear Materials, 2010, 405, 235-243.	1.3	75
58	Influence of the U_3O_7 domain structure on cracking during the oxidation of UO_2 . Journal of Nuclear Materials, 2010, 402, 167-172.	1.3	20
59	Internal Interface Strains Effects on UO_2/U_3O_7 Oxidation Behaviour. Defect and Diffusion Forum, 2010, 297-301, 519-524.	0.4	6
60	In situ Characterization of UO_2 Microstructure Changes During an Annealing Test in an Environmental Scanning Electron Microscope. Materials Research Society Symposia Proceedings, 2009, 1215, 1.	0.1	0
61	Deuterium inventory in Tore Supra: reconciling particle balance and post-mortem analysis. Nuclear Fusion, 2009, 49, 075011.	1.6	53
62	Neutron Diffraction Study of the Structural Changes Occurring During the Low Temperature Oxidation of UO_2 . Materials Research Society Symposia Proceedings, 2009, 1215, 1.	0.1	3
63	Modelling of the spent fuel oxidation: Toward the operational model. Journal of Nuclear Materials, 2009, 395, 89-98.	1.3	1
64	Synthesis and characterization of nanometric powders of UO_{2+x} , $(Th,U)O_{2+x}$ and $(La,U)O_{2+x}$. Journal of Solid State Chemistry, 2009, 182, 2591-2597.	1.4	29
65	On the origin of the sigmoid shape in the UO_2 oxidation weight gain curves. Journal of the European Ceramic Society, 2009, 29, 2791-2798.	2.8	29
66	SIMS characterisation of actinide isotopes in irradiated nuclear fuel. Journal of Nuclear Materials, 2009, 385, 99-102.	1.3	16
67	ADAGIO technique: From UO_2 fuels to MOX fuels. Journal of Nuclear Materials, 2009, 385, 137-141.	1.3	3
68	Neutron Diffraction Study of the in Situ Oxidation of UO_2 . Inorganic Chemistry, 2009, 48, 7585-7592.	1.9	146
69	HIGH BURNUP CHANGES IN UO_2 FUELS IRRADIATED UP TO 83 GWD/T IN M_5 CLADDINGS. Nuclear Engineering and Technology, 2009, 41, 155-162.	1.1	32
70	Detailed characterisations of high burn-up structures in oxide fuels. Journal of Nuclear Materials, 2008, 372, 318-339.	1.3	140
71	A method for the quantification of total xenon concentration in irradiated nuclear fuel with SIMS and EPMA. Nuclear Instruments & Methods in Physics Research B, 2008, 266, 147-154.	0.6	18
72	Study of structural material resulting from the nuclear fuel cycle using SEM-WDX, EPMA and SIMS techniques. Mikrochimica Acta, 2008, 161, 355-362.	2.5	5

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73	Evidence of two gas release kinetics during the oxidation of an irradiated PWR UO ₂ fuel. Journal of Nuclear Materials, 2008, 378, 1-8.	1.3	16
74	Assessment of the Nd/U ratio for the quantification of neodymium in UO ₂ . Applied Surface Science, 2008, 255, 863-865.	3.1	3
75	Interpretation of the molybdenum behaviour in irradiated UO ₂ using a point defect approach. Nuclear Instruments & Methods in Physics Research B, 2008, 266, 3018-3022.	0.6	5
76	Oxygen Lattice Distortions and U Oxidation State in UO _{2+x} Fluorite Structures. Materials Research Society Symposia Proceedings, 2008, 1124, 1.	0.1	1
77	A New Criterion for the Degradation of a Defective Spent Fuel Rod under Dry Storage Conditions Based on Nuclear Ceramic Cracking. Materials Research Society Symposia Proceedings, 2008, 1124, 1.	0.1	2
78	Behavior of a Defective Nuclear Fuel Rod in Dry Storage Conditions Studied with a New Experimental Setup. Nuclear Technology, 2008, 163, 252-260.	0.7	5
79	An improved model to evaluate the oxidation kinetics of uranium dioxide during dry storage. Journal of Nuclear Materials, 2007, 362, 402-410.	1.3	20
80	Characterisation of irradiated nuclear fuel with SIMS. Applied Surface Science, 2006, 252, 7048-7050.	3.1	14
81	Specific outcomes of the research on the spent fuel long-term evolution in interim dry storage and deep geological disposal. Journal of Nuclear Materials, 2006, 352, 246-253.	1.3	52
82	A detailed study of UO ₂ to U ₃ O ₈ oxidation phases and the associated rate-limiting steps. Journal of Nuclear Materials, 2006, 355, 10-20.	1.3	94
83	Fission Gas Bubbles Characterisation in Irradiated UO ₂ Fuel by SEM, EPMA and SIMS. Mikrochimica Acta, 2006, 155, 183-187.	2.5	25
84	Modeling of Spent Fuel Oxidation at Low Temperature. Materials Research Society Symposia Proceedings, 2006, 985, 1.	0.1	1
85	Oxidation kinetics of high burn-up UOX and MOX fuel : the influence of the grain boundaries. Materials Research Society Symposia Proceedings, 2006, 932, 1.	0.1	2
86	A New Apparatus for Determination of the Free Volume of a Fuel Rod Using the Double Expansion Method. Nuclear Technology, 2005, 149, 14-21.	0.7	3
87	Electrical Conductivity and Thermoelectric Power of Uranium Dioxide. Journal of the American Ceramic Society, 2005, 88, 604-611.	1.9	54
88	Characterisation of uranium vacancies in hyper stoichiometric uranium dioxide. Journal of the European Ceramic Society, 2005, 25, 2683-2686.	2.8	8
89	Heat capacity anomaly in UO ₂ in the vicinity of 1300K: an improved description based on high resolution X-ray and neutron powder diffraction studies. Journal of Physics and Chemistry of Solids, 2005, 66, 823-831.	1.9	42
90	Contribution of the synchrotron diffraction study of the oxidation of uranium dioxide at 250°C. European Physical Journal Special Topics, 2004, 118, 127-134.	0.2	5

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91	Influence of Defect Interactions on Diffusion Processes in UO _{2+x} : a Key Issue for Understanding the Behaviour of Spent Nuclear Fuel.. Materials Research Society Symposia Proceedings, 2004, 824, 507.	0.1	3
92	Measurement of xenon in uranium dioxide (UO ₂) with SIMS. Nuclear Instruments & Methods in Physics Research B, 2004, 215, 545-551.	0.6	37
93	Detection of Gas Bubble by SIMS in Irradiated Nuclear Fuel. Mikrochimica Acta, 2004, 145, 91-94.	2.5	20
94	Chemical diffusion in uranium dioxide " influence of defect interactions. Journal of Nuclear Materials, 2004, 325, 202-209.	1.3	35
95	Thermal variation of the optical absorption of UO ₂ : determination of the small polaron self-energy. Journal of Nuclear Materials, 2004, 328, 46-54.	1.3	47
96	A new shielded SIMS instrument for analysis of highly radioactive materials. Applied Surface Science, 2003, 203-204, 673-678.	3.1	23
97	Post-irradiation examinations of THERMHET composite fuels for transmutation. Journal of Nuclear Materials, 2003, 320, 117-125.	1.3	13
98	Behaviour of fission gases in an irradiated nuclear fuel under ± external irradiation. Journal of Nuclear Materials, 2003, 321, 324-330.	1.3	3
99	A New Modelling of the Kinetics of Uranium Dioxide Oxidation in Air. Materials Research Society Symposia Proceedings, 2003, 807, 134.	0.1	1
100	Synchrotron Diffraction Study of the Isothermal Oxidation of Uranium Dioxide at 250°C. Materials Research Society Symposia Proceedings, 2003, 802, 3.	0.1	2
101	High magnification SEM observations for two types of granularity in a high burnup PWR fuel rim. Journal of Nuclear Materials, 1998, 257, 78-87.	1.3	45
102	Inelastic neutron scattering study of proton dynamics in Ca(OH) ₂ at 20 K. Chemical Physics, 1995, 197, 81-90.	0.9	14
103	Structural and thermodynamic evidence of a change in thermal motion of hydrogen atoms in Ca(OH) ₂ at low temperature. Journal of Physics and Chemistry of Solids, 1994, 55, 161-166.	1.9	10
104	Hydrogen thermal motion in calcium hydroxide: Ca(OH) ₂ . Acta Crystallographica Section B: Structural Science, 1993, 49, 812-817.	1.8	115
105	Structural transformations preparing the dehydration of Ca(OH) ₂ . Phase Transitions, 1991, 31, 283-286.	0.6	3
106	A Thermodynamic Approach of the Mechano-Chemical Coupling during the Oxidation of Uranium Dioxide. Defect and Diffusion Forum, 0, 289-292, 447-454.	0.4	9
107	Evidence of a Biphasic Domain in the UO ₂ -Nd ₂ O ₃ Diagram at Room Temperature: a Proof for a Miscibility Gap in UO ₂ -Nd ₂ O ₃ ; Phase Diagram ?. Solid State Phenomena, 0, 172-174, 624-629.	0.3	7
108	Specific Aspects of Internal Corrosion of Nuclear Clad Made of Zircaloy. Defect and Diffusion Forum, 0, 323-325, 227-232.	0.4	4