

# Michael Rieth

## List of Publications by Year in Descending Order

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The third column is the impact factor (IF) of the journal, and the fourth column is the number of citations of the article.

174  
papers

5,689  
citations

43  
h-index

70  
g-index

185  
ext. papers

6,434  
ext. citations

2.6  
avg, IF

5.56  
L-index

#	Paper	IF	Citations
174	On the mechanical alloying of novel austenitic dual-precipitation strengthened steels. <i>Materials and Design</i> , <b>2022</b> , 213, 110316	8.1	
173	Recent progress in the assessment of irradiation effects for in-vessel fusion materials: tungsten and copper alloys. <i>Nuclear Fusion</i> , <b>2022</b> , 62, 026045	3.3	3
172	Effect of neutron irradiation on ductility of tungsten foils developed for tungsten-copper laminates. <i>Nuclear Materials and Energy</i> , <b>2022</b> , 30, 101133	2.1	1
171	Materials for in-vessel components. <i>Fusion Engineering and Design</i> , <b>2022</b> , 174, 112994	1.7	2
170	Charpy impact tests of tungsten fiber-reinforced composite from 150 °C to 1000 °C. <i>Materials Letters</i> , <b>2022</b> , 311, 131526	3.3	1
169	Nanoscale insights into the corrosion of EUROFER by lithium ceramics. <i>Corrosion Science</i> , <b>2022</b> , 199, 1101690	1.7	1
168	S/TEM examination and nanomechanical response of W-Eurofer joints brazed with Cu interlayers. <i>Nuclear Materials and Energy</i> , <b>2022</b> , 31, 101155	2.1	
167	Tungsten-tantalum alloys for fusion reactor applications. <i>Journal of Nuclear Materials</i> , <b>2022</b> , 566, 153740	3.3	2
166	Experimental Investigation of EU-DEMO Breeding Blanket First Wall Mock-Ups in Support of the Manufacturing and Material Development Programmes. <i>Energies</i> , <b>2021</b> , 14, 7580	3.1	0
165	Irradiation hardening and ductility loss of Eurofer97 steel variants after neutron irradiation to ITER-TBM relevant conditions. <i>Fusion Engineering and Design</i> , <b>2021</b> , 173, 112935	1.7	1
164	New insights into microstructure of neutron-irradiated tungsten. <i>Scientific Reports</i> , <b>2021</b> , 11, 7572	4.9	9
163	Additive manufacturing technologies for EUROFER97 components. <i>Journal of Nuclear Materials</i> , <b>2021</b> , 548, 152859	3.3	4
162	Impact of materials technology on the breeding blanket design   Recent progress and case studies in materials technology. <i>Fusion Engineering and Design</i> , <b>2021</b> , 166, 112275	1.7	5
161	Advances in Additive Manufacturing of fusion materials. <i>Fusion Engineering and Design</i> , <b>2021</b> , 167, 112309	1.7	3
160	Plastic deformation in advanced tungsten-based alloys for fusion applications studied by mechanical testing and TEM. <i>International Journal of Refractory Metals and Hard Materials</i> , <b>2021</b> , 95, 105409	4.1	8
159	Mechanical properties of tungsten: Recent research on modified tungsten materials in Japan. <i>Journal of Nuclear Materials</i> , <b>2021</b> , 543, 152506	3.3	28
158	Comparison of K-doped and pure cold-rolled tungsten sheets: Tensile properties and brittle-to-ductile transition temperatures. <i>Journal of Nuclear Materials</i> , <b>2021</b> , 544, 152664	3.3	7

157	Technology readiness assessment of materials for DEMO in-vessel applications. <i>Journal of Nuclear Materials</i> , <b>2021</b> , 550, 152906	3.3	4
156	Technological aspects in blanket design: Effects of micro-alloying and thermo-mechanical treatments of EUROFER97 type steels after neutron irradiation. <i>Fusion Engineering and Design</i> , <b>2021</b> , 168, 112645	1.7	4
155	Microstructure and hardening induced by neutron irradiation in single crystal, ITER specification and cold rolled tungsten. <i>International Journal of Refractory Metals and Hard Materials</i> , <b>2021</b> , 98, 105522	4.1	10
154	Neutron irradiation tolerance of potassium-doped and rhenium-alloyed tungsten. <i>Journal of Nuclear Materials</i> , <b>2021</b> , 553, 153009	3.3	8
153	DEMO structural materials qualification and development. <i>Fusion Engineering and Design</i> , <b>2021</b> , 170, 112513	1.7	1
152	Microstructure and precipitation behavior of advanced RAFM steels for high-temperature applications on fusion reactors. <i>Materials Characterization</i> , <b>2021</b> , 180, 111443	3.9	2
151	Evolution of microstructure in neutron irradiated cold rolled tungsten and its correlation with hardness. <i>Fusion Engineering and Design</i> , <b>2021</b> , 172, 112784	1.7	3
150	Microstructural features in additively manufactured EUROFER97 components. <i>Fusion Engineering and Design</i> , <b>2021</b> , 173, 112813	1.7	0
149	Post-irradiation microstructural examination of EUROFER-ODS steel irradiated at 300°C and 400°C. <i>Journal of Nuclear Materials</i> , <b>2021</b> , 557, 153259	3.3	1
148	Technological Processes for Steel Applications in Nuclear Fusion. <i>Applied Sciences (Switzerland)</i> , <b>2021</b> , 11, 11653	2.6	0
147	Fabrication of HCPB breeding blanket components using the additive manufacturing processes of selective laser melting and cold spray. <i>Fusion Engineering and Design</i> , <b>2020</b> , 160, 112026	1.7	1
146	Correlation of microstructural and mechanical properties of neutron irradiated EUROFER97 steel. <i>Journal of Nuclear Materials</i> , <b>2020</b> , 538, 152231	3.3	11
145	The brittle-to-ductile transition in cold-rolled tungsten sheets: the rate-limiting mechanism of plasticity controlling the BDT in ultrafine-grained tungsten. <i>Journal of Materials Science</i> , <b>2020</b> , 55, 12314	4.3	9
144	Elucidating the microstructure of tungsten composite materials produced by powder injection molding. <i>Nuclear Materials and Energy</i> , <b>2020</b> , 24, 100766	2.1	1
143	Fracture behavior of tungsten-based composites exposed to steady-state/transient hydrogen plasma. <i>Nuclear Fusion</i> , <b>2020</b> , 60, 046029	3.3	7
142	Tungsten modified by potassium doping and rhenium addition for fusion reactor applications. <i>Fusion Engineering and Design</i> , <b>2020</b> , 152, 111445	1.7	20
141	Cavity formation and hardness change in He implanted EUROFER97 and EU-ODS EUROFER. <i>Nuclear Materials and Energy</i> , <b>2020</b> , 22, 100717	2.1	1
140	The brittle-to-ductile transition in cold-rolled tungsten sheets: On the loss of room-temperature ductility after annealing and the phenomenon of 45°C embrittlement. <i>International Journal of Refractory Metals and Hard Materials</i> , <b>2020</b> , 93, 105347	4.1	9

139	The brittle-to-ductile transition in cold-rolled tungsten sheets: Contributions of grain and subgrain boundaries to the enhanced ductility after pre-deformation. <i>Nuclear Materials and Energy</i> , <b>2020</b> , 25, 100769	2.1	3
138	Effects of neutron irradiation on the brittle to ductile transition in single crystal tungsten. <i>Journal of Nuclear Materials</i> , <b>2019</b> , 527, 151799	3.3	19
137	Tensile and impact properties of tungsten-rhenium alloy for plasma-facing components in fusion reactor. <i>Fusion Engineering and Design</i> , <b>2019</b> , 148, 111323	1.7	15
136	Microstructural investigation of an extruded austenitic oxide dispersion strengthened steel containing a carbon-containing process control agent. <i>Journal of Nuclear Materials</i> , <b>2019</b> , 516, 335-346	3.3	10
135	Improvement of impact properties of tungsten by potassium doping. <i>Fusion Engineering and Design</i> , <b>2019</b> , 140, 48-61	1.7	16
134	Manufacturing, high heat flux testing and post mortem analyses of a W-PIM mock-up. <i>Nuclear Materials and Energy</i> , <b>2019</b> , 20, 100688	2.1	1
133	Long-term stability of the microstructure of austenitic ODS steel rods produced with a carbon-containing process control agent. <i>Journal of Nuclear Materials</i> , <b>2019</b> , 523, 111-120	3.3	4
132	High pulse number thermal shock testing of tungsten alloys produced by powder injection molding. <i>Nuclear Materials and Energy</i> , <b>2019</b> , 20, 100680	2.1	7
131	Mechanical properties and microstructure characterization of Eurofer97 steel variants in EUROfusion program. <i>Fusion Engineering and Design</i> , <b>2019</b> , 146, 2227-2232	1.7	11
130	Behavior of tungsten under irradiation and plasma interaction. <i>Journal of Nuclear Materials</i> , <b>2019</b> , 519, 334-368	3.3	73
129	European materials development: Results and perspective. <i>Fusion Engineering and Design</i> , <b>2019</b> , 146, 1300-1307	1.7	30
128	Deformation behaviour and microstructural evolution of EUROFER97-2 under low cycle fatigue conditions. <i>Materials Characterization</i> , <b>2019</b> , 158, 109943	3.9	3
127	Tensile properties of baseline and advanced tungsten grades for fusion applications. <i>International Journal of Refractory Metals and Hard Materials</i> , <b>2018</b> , 75, 153-162	4.1	48
126	Radiation damage studies in fusion reactor steels by means of small-angle neutron scattering (SANS). <i>Physica B: Condensed Matter</i> , <b>2018</b> , 551, 407-412	2.8	4
125	TEM characterization on new 9% Cr advanced steels thermomechanical treated after tempering. <i>Journal of Nuclear Materials</i> , <b>2018</b> , 500, 1-10	3.3	16
124	Processing of complex near-net-shaped tungsten parts by PIM. <i>Nuclear Materials and Energy</i> , <b>2018</b> , 16, 71-75	2.1	9
123	Development of EUROFER97 database and material property handbook. <i>Fusion Engineering and Design</i> , <b>2018</b> , 135, 9-14	1.7	21
122	A review of impact properties of tungsten materials. <i>Fusion Engineering and Design</i> , <b>2018</b> , 135, 196-203	1.7	18

121	Master Curve Fracture Toughness Characterization of Eurofer97 Using Miniature Multi-Notch Bend Bar Specimens for Fusion Applications <b>2018</b> ,		2
120	3D Structural Analysis of Selected High-Temperature Materials. <i>Praktische Metallographie/Practical Metallography</i> , <b>2018</b> , 55, 424-446	0.3	
119	Micro-structural effects of irradiation temperature and helium content in neutron irradiated B-alloyed Eurofer97-1 steel. <i>Nuclear Materials and Energy</i> , <b>2018</b> , 17, 40-47	2.1	5
118	Interfacial characterization by TEM and nanoindentation of W-Eurofer brazed joints for the first wall component of the DEMO fusion reactor. <i>Materials Characterization</i> , <b>2018</b> , 142, 162-169	3.9	11
117	Investigation of precipitate in an austenitic ODS steel containing a carbon-rich process control agent. <i>Nuclear Materials and Energy</i> , <b>2018</b> , 15, 237-243	2.1	10
116	Improvement of EUROFER's mechanical properties by optimized chemical compositions and thermo-mechanical treatments. <i>Nuclear Materials and Energy</i> , <b>2018</b> , 16, 88-94	2.1	8
115	Measurements and controls implementation for WEST. <i>Fusion Engineering and Design</i> , <b>2017</b> , 123, 1029-1032	1.7	5
114	Production, microstructure and mechanical properties of two different austenitic ODS steels. <i>Journal of Nuclear Materials</i> , <b>2017</b> , 487, 348-361	3.3	27
113	Development of advanced high heat flux and plasma-facing materials. <i>Nuclear Fusion</i> , <b>2017</b> , 57, 092007	3.3	137
112	Development of next generation tempered and ODS reduced activation ferritic/martensitic steels for fusion energy applications. <i>Nuclear Fusion</i> , <b>2017</b> , 57, 092005	3.3	119
111	Choice of a low operating temperature for the DEMO EUROFER97 divertor cassette. <i>Fusion Engineering and Design</i> , <b>2017</b> , 124, 655-658	1.7	25
110	Advanced materials for a damage resilient divertor concept for DEMO: Powder-metallurgical tungsten-fibre reinforced tungsten. <i>Fusion Engineering and Design</i> , <b>2017</b> , 124, 964-968	1.7	32
109	Ductilisation of tungsten (W): Tungsten laminated composites. <i>International Journal of Refractory Metals and Hard Materials</i> , <b>2017</b> , 69, 66-109	4.1	48
108	Assessment of industrial nitriding processes for fusion steel applications. <i>Nuclear Materials and Energy</i> , <b>2017</b> , 13, 90-98	2.1	1
107	Ductilisation of tungsten (W): On the increase of strength AND room-temperature tensile ductility through cold-rolling. <i>International Journal of Refractory Metals and Hard Materials</i> , <b>2017</b> , 64, 261-278	4.1	37
106	Irradiation effects in tungsten-copper laminate composite. <i>Journal of Nuclear Materials</i> , <b>2016</b> , 481, 134-146	3.5	22
105	Identification of Laves- and Z-phase formed in 9%Cr ferritic alloy after long-term thermal treatment <b>2016</b> , 1056-1057		
104	Conceptual design studies for the European DEMO divertor: Rationale and first results. <i>Fusion Engineering and Design</i> , <b>2016</b> , 109-111, 1598-1603	1.7	93

103	Progress in the engineering design and assessment of the European DEMO first wall and divertor plasma facing components. <i>Fusion Engineering and Design</i> , <b>2016</b> , 109-111, 917-924	1.7	57
102	European DEMO divertor target: Operational requirements and material-design interface. <i>Nuclear Materials and Energy</i> , <b>2016</b> , 9, 171-176	2.1	92
101	Numerical exploration into the potential of tungsten reinforced CuCrZr matrix composites. <i>Journal of Nuclear Materials</i> , <b>2016</b> , 470, 13-29	3.3	10
100	Ductilisation of tungsten (W) through cold-rolling: R-curve behaviour. <i>International Journal of Refractory Metals and Hard Materials</i> , <b>2016</b> , 58, 22-33	4.1	35
99	Manufacturing and characterization of PIM-W materials as plasma facing materials. <i>Physica Scripta</i> , <b>2016</b> , T167, 014056	2.6	2
98	Improvement of reduced activation 9%Cr steels by ausforming. <i>Nuclear Materials and Energy</i> , <b>2016</b> , 6, 12-17	2.1	33
97	Overview of the design approach and prioritization of R&D activities towards an EU DEMO. <i>Fusion Engineering and Design</i> , <b>2016</b> , 109-111, 1464-1474	1.7	158
96	Effect of neutron irradiation on the microstructure of tungsten. <i>Nuclear Materials and Energy</i> , <b>2016</b> , 9, 480-483	2.1	45
95	Materials for DEMO and reactor applications Boundary conditions and new concepts. <i>Physica Scripta</i> , <b>2016</b> , T167, 014002	2.6	68
94	Ductilisation of tungsten (W): On the shift of the brittle-to-ductile transition (BDT) to lower temperatures through cold rolling. <i>International Journal of Refractory Metals and Hard Materials</i> , <b>2016</b> , 54, 351-369	4.1	70
93	Recent status and improvement of reduced-activation ferritic-martensitic steels for high-temperature service. <i>Journal of Nuclear Materials</i> , <b>2016</b> , 479, 515-523	3.3	53
92	Microstructural anisotropy of ferritic ODS alloys after different production routes. <i>Fusion Engineering and Design</i> , <b>2015</b> , 98-99, 1986-1990	1.7	12
91	Tungsten laminates made of ultrafine-grained (UFG) tungsten foil / Ageing of tungsten/titanium (W/Ti) laminates. <i>International Journal of Refractory Metals and Hard Materials</i> , <b>2015</b> , 51, 264-274	4.1	14
90	Testing candidate interlayers for an enhanced water-cooled divertor target. <i>Fusion Engineering and Design</i> , <b>2015</b> , 98-99, 1323-1327	1.7	1
89	Mechanical and microstructural investigations of tungsten and doped tungsten materials produced via powder injection molding. <i>Nuclear Materials and Energy</i> , <b>2015</b> , 3-4, 22-31	2.1	46
88	Enhancing the DEMO divertor target by interlayer engineering. <i>Fusion Engineering and Design</i> , <b>2015</b> , 98-99, 1216-1220	1.7	24
87	The nature of the brittle-to-ductile transition of ultra fine grained tungsten (W) foil. <i>International Journal of Refractory Metals and Hard Materials</i> , <b>2015</b> , 50, 9-15	4.1	44
86	Microstructural and mechanical characterization of annealed tungsten (W) and potassium-doped tungsten foils. <i>International Journal of Refractory Metals and Hard Materials</i> , <b>2015</b> , 48, 145-149	4.1	24

85	Neutron diffraction stress determination in W-laminates for structural divertor applications. <i>Nuclear Materials and Energy</i> , <b>2015</b> , 3-4, 37-42	2.1	2
84	Tungsten (W) Laminate Pipes for Innovative High Temperature Energy Conversion Systems. <i>Advanced Engineering Materials</i> , <b>2015</b> , 17, 491-501	3.5	26
83	Optimization of growth parameters for growth of high quality heteroepitaxial 3C <sub>2</sub> BiC films at 1200°C. <i>Thin Solid Films</i> , <b>2015</b> , 577, 88-93	2.2	10
82	Materials R&D for a timely DEMO: Key findings and recommendations of the EU Roadmap Materials Assessment Group. <i>Fusion Engineering and Design</i> , <b>2014</b> , 89, 1586-1594	1.7	99
81	Developing structural, high-heat flux and plasma facing materials for a near-term DEMO fusion power plant: The EU assessment. <i>Journal of Nuclear Materials</i> , <b>2014</b> , 455, 277-291	3.3	172
80	Effect of helium implantation on mechanical properties of EUROFER97 evaluated by nanoindentation. <i>Journal of Nuclear Materials</i> , <b>2014</b> , 448, 301-309	3.3	15
79	Overview of the Structural Materials Program for Fusion Reactors under EFDA. <i>Fusion Science and Technology</i> , <b>2014</b> , 66, 38-45	1.1	6
78	Creep-Fatigue Interaction in Eurofer 3 Electron Beam Welds. <i>Fusion Science and Technology</i> , <b>2014</b> , 66, 131-135	1.1	0
77	Investigation of European tungsten materials exposed to high heat flux H/He neutral beams. <i>Journal of Nuclear Materials</i> , <b>2013</b> , 442, S256-S260	3.3	20
76	Tungsten foil laminate for structural divertor applications Joining of tungsten foils. <i>Journal of Nuclear Materials</i> , <b>2013</b> , 436, 47-55	3.3	28
75	A brief summary of the progress on the EFDA tungsten materials program. <i>Journal of Nuclear Materials</i> , <b>2013</b> , 442, S173-S180	3.3	63
74	Recent progress in R&D on tungsten alloys for divertor structural and plasma facing materials. <i>Journal of Nuclear Materials</i> , <b>2013</b> , 442, S181-S189	3.3	222
73	Characterization of ODS (Oxide Dispersion Strengthened) Eurofer/Eurofer dissimilar electron beam welds. <i>Journal of Nuclear Materials</i> , <b>2013</b> , 442, S552-S556	3.3	10
72	Microstructure and mechanical properties of a W <sub>10</sub> wt.%Y <sub>2</sub> O <sub>3</sub> composite produced by sintering and hot forging. <i>Journal of Nuclear Materials</i> , <b>2013</b> , 442, S225-S228	3.3	35
71	Investigation on different oxides as candidates for nano-sized ODS particles in reduced-activation ferritic (RAF) steels. <i>Journal of Nuclear Materials</i> , <b>2013</b> , 442, 444-448	3.3	31
70	A fail-safe and cost effective fabrication route for blanket First Walls. <i>Journal of Nuclear Materials</i> , <b>2013</b> , 442, 538-541	3.3	3
69	Investigations of dissimilar welds of the high temperature steels P91 and PM2000. <i>Fusion Engineering and Design</i> , <b>2013</b> , 88, 2539-2542	1.7	7
68	KATHELO: A new high heat flux component testing facility. <i>Fusion Engineering and Design</i> , <b>2013</b> , 88, 854-857	1.7	6

67	Charpy impact properties of pure tungsten plate material in as-received and recrystallized condition (1h at 2000°C (2273K)). <i>Journal of Nuclear Materials</i> , <b>2013</b> , 442, S204-S207	3-3	33
66	Tungsten foil laminate for structural divertor applications I Tensile test properties of tungsten foil. <i>Journal of Nuclear Materials</i> , <b>2013</b> , 434, 357-366	3-3	45
65	Recent progress in research on tungsten materials for nuclear fusion applications in Europe. <i>Journal of Nuclear Materials</i> , <b>2013</b> , 432, 482-500	3-3	494
64	TEM study of mechanically alloyed ODS steel powder. <i>Journal of Nuclear Materials</i> , <b>2012</b> , 428, 165-169	3-3	11
63	Tungsten foil laminate for structural divertor applications I Basics and outlook. <i>Journal of Nuclear Materials</i> , <b>2012</b> , 423, 1-8	3-3	70
62	Tungsten foil laminate for structural divertor applications I Analyses and characterisation of tungsten foil. <i>Journal of Nuclear Materials</i> , <b>2012</b> , 424, 197-203	3-3	54
61	Optimization and limitations of known DEMO divertor concepts. <i>Fusion Engineering and Design</i> , <b>2012</b> , 87, 718-721	1-7	33
60	The Impact of Refractory Material Properties on the Helium Cooled Divertor Design. <i>Fusion Science and Technology</i> , <b>2012</b> , 61, 381-384	1-1	24
59	Technology Developments at KIT Towards a Magnetic Confinement Fusion Power Plant. <i>Fusion Science and Technology</i> , <b>2012</b> , 61, 64-69	1-1	5
58	Deep drawing of tungsten plates for structural divertor applications in future fusion devices. <i>Fusion Engineering and Design</i> , <b>2011</b> , 86, 2949-2953	1-7	11
57	W-Based Alloys for Advanced Divertor Designs: Options and Environmental Impact of State-of-the-Art Alloys. <i>Fusion Science and Technology</i> , <b>2011</b> , 60, 185-189	1-1	25
56	Innovative materials for Gen IV systems and transmutation facilities: The cross-cutting research project GETMAT. <i>Nuclear Engineering and Design</i> , <b>2011</b> , 241, 3514-3520	1-8	32
55	Review on the EFDA work programme on nano-structured ODS RAF steels. <i>Journal of Nuclear Materials</i> , <b>2011</b> , 417, 149-153	3-3	60
54	Review of candidate welding processes of RAFM steels for ITER test blanket modules and DEMO. <i>Journal of Nuclear Materials</i> , <b>2011</b> , 417, 43-50	3-3	56
53	Development of welding technologies for the manufacturing of European Tritium Breeder blanket modules. <i>Journal of Nuclear Materials</i> , <b>2011</b> , 417, 36-42	3-3	26
52	Creep-fatigue interaction and related structure property correlations of EUROFER97 steel at 550 °C by decoupling creep and fatigue load. <i>Journal of Nuclear Materials</i> , <b>2011</b> , 417, 16-19	3-3	12
51	Modelling structural and plasma facing materials for fusion power plants: Recent advances and outstanding issues in the EURATOM fusion materials programme. <i>Journal of Nuclear Materials</i> , <b>2011</b> , 417, 1042-1049	3-3	8
50	Review on the EFDA programme on tungsten materials technology and science. <i>Journal of Nuclear Materials</i> , <b>2011</b> , 417, 463-467	3-3	139

49	FeCrV ferritic steels for possible nuclear applications. <i>Journal of Nuclear Materials</i> , <b>2011</b> , 409, 140-146	3.3	6
48	Low activation steels welding with PWHT and coating for ITER test blanket modules and DEMO. <i>Journal of Nuclear Materials</i> , <b>2011</b> , 409, 156-162	3.3	14
47	First-Principles Modeling of Tungsten-Based Alloys for Fusion Power Plant Applications. <i>Key Engineering Materials</i> , <b>2011</b> , 465, 15-20	0.4	13
46	Tungsten as a Structural Divertor Material. <i>Advances in Science and Technology</i> , <b>2010</b> , 73, 11-21	0.1	53
45	Influence of microstructure and notch fabrication on impact bending properties of tungsten materials. <i>International Journal of Refractory Metals and Hard Materials</i> , <b>2010</b> , 28, 679-686	4.1	78
44	High heat flux components Readiness to proceed from near term fusion systems to power plants. <i>Fusion Engineering and Design</i> , <b>2010</b> , 85, 93-108	1.7	99
43	Specific welds for test blanket modules. <i>Journal of Nuclear Materials</i> , <b>2009</b> , 386-388, 471-474	3.3	25
42	Diffusion weld study for Test Blanket Module fabrication. <i>Fusion Engineering and Design</i> , <b>2009</b> , 84, 1602-1605	1.3	13
41	Transmutation and activation analysis for divertor materials in a HCLL-type fusion power reactor. <i>Journal of Nuclear Materials</i> , <b>2009</b> , 386-388, 789-792	3.3	9
40	European cross-cutting research on structural materials for Generation IV and transmutation systems. <i>Journal of Nuclear Materials</i> , <b>2009</b> , 392, 316-323	3.3	53
39	He-Cooled Divertor Development Towards DEMO. <i>Fusion Science and Technology</i> , <b>2009</b> , 56, 1013-1017	1.1	11
38	Fracture Behaviour of Tungsten Based Alloys Depending on Microstructure and Notch Fabrication Method. <i>Fusion Science and Technology</i> , <b>2009</b> , 56, 1018-1022	1.1	16
37	Impact Bending Tests on Selected Refractory Materials. <i>Advanced Materials Research</i> , <b>2008</b> , 59, 101-104	0.5	20
36	Formation and growth of complex precipitates in 316L austenitic steel during long-term annealing experiments. <i>Journal of Materials Science</i> , <b>2008</b> , 43, 2541-2549	4.3	2
35	Fissile core and Tritium-Breeding Blanket: structural materials and their requirements. <i>Comptes Rendus Physique</i> , <b>2008</b> , 9, 287-302	1.4	62
34	Modelling irradiation effects in fusion materials. <i>Fusion Engineering and Design</i> , <b>2007</b> , 82, 2413-2421	1.7	40
33	Precipitation in AISI 316L(N) during creep tests at 550 and 600°C up to 10 years. <i>Journal of Nuclear Materials</i> , <b>2007</b> , 362, 132-138	3.3	68
32	A comprising steady-state creep model for the austenitic AISI 316 L(N) steel. <i>Journal of Nuclear Materials</i> , <b>2007</b> , 367-370, 915-919	3.3	21

31	Development of a helium-cooled divertor: Material choice and technological studies. <i>Journal of Nuclear Materials</i> , <b>2007</b> , 367-370, 1416-1421	3.3	133
30	The European effort towards the development of a demo structural material: Irradiation behaviour of the European reference RAFM steel EUROFER. <i>Fusion Engineering and Design</i> , <b>2006</b> , 81, 917-923	1.7	38
29	Displacement cascades in FeCr: A molecular dynamics study. <i>Journal of Nuclear Materials</i> , <b>2006</b> , 349, 119-132	3.3	97
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27	Creep strength of reduced activation ferritic/martensitic steel Eurofer97. <i>Fusion Engineering and Design</i> , <b>2005</b> , 75-79, 1003-1008	1.7	28
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24	Self-organizing processes in connection with metastable nanocluster states. <i>Journal of Vacuum Science and Technology A: Vacuum, Surfaces and Films</i> , <b>2004</b> , 22, 1433-1438	2.9	
23	Development of a helium-cooled divertor concept: design-related requirements on materials and fabrication technology. <i>Journal of Nuclear Materials</i> , <b>2004</b> , 329-333, 1594-1598	3.3	111
22	Welcome to the Journal of Computational and Theoretical Nanoscience. <i>Journal of Computational and Theoretical Nanoscience</i> , <b>2004</b> , 1, 1-2	0.3	3
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10	Influence of helium on impact properties of reduced-activation ferritic/martensitic Cr-steels. <i>Journal of Nuclear Materials</i> , <b>1999</b> , 271-272, 450-454	3-3	43
9	Embrittlement behaviour of different international low activation alloys after neutron irradiation. <i>Journal of Nuclear Materials</i> , <b>1998</b> , 258-263, 1147-1152	3-3	73
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