

# Masataka Nakahira

## List of Publications by Year in descending order

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#	ARTICLE	IF	CITATIONS
1	Evaluation of Mass Production Results of Cryogenic Structural Stainless Steels for ITER Toroidal Field Coils. IEEE Transactions on Applied Superconductivity, 2022, 32, 1-5.	1.7	2
2	Completion of the First ITER TF Coil in the Second Manufacturing Line in Japan. IEEE Transactions on Applied Superconductivity, 2022, 32, 1-6.	1.7	2
3	Completion of the first ITER toroidal field coil in Japan. Nuclear Fusion, 2021, 61, 116044.	3.5	5
4	Progress of ITER TF Coil Fabrication in Japan. IEEE Transactions on Applied Superconductivity, 2020, 30, 1-6.	1.7	10
5	Development of ITER Toroidal Field Coil Winding Packs. TEION KOGAKU (Journal of Cryogenics and Superconductivity Society of Japan), 2020, 55, 400-408.	0.1	6
6	Development of ITER TF Coil Assembly Technique, Integration of Winding Pack into Coil Case. TEION KOGAKU (Journal of Cryogenics and Superconductivity Society of Japan), 2020, 55, 400-408.	0.1	6
7	Development of Gap-filling Impregnation Method of ITER TF Coils. TEION KOGAKU (Journal of Cryogenics and Superconductivity Society of Japan), 2020, 55, 400-408.	0.1	6
8	Completion of the first ITER toroidal field coil structure. Nuclear Fusion, 2019, 59, 086039.	3.5	7
9	Development of ITER TF Coil Winding Pack (WP) and Qualification for Assembling WP and Coil Case in Japan. IEEE Transactions on Applied Superconductivity, 2019, 29, 1-5.	1.7	7
10	Progress of ITER TF Coil Case Fabrication in Japan. IEEE Transactions on Applied Superconductivity, 2018, 28, 1-5.	1.7	4
11	Correlation between the Fracture Toughness of Austenite Stainless Steel and Stability of the Austenite Phase in Cryogenic State. TEION KOGAKU (Journal of Cryogenics and Superconductivity Society of Japan), 2020, 55, 400-408.	0.1	6
12	Progress in Procurement of ITER Toroidal Field Coil in Japan. IEEE Transactions on Applied Superconductivity, 2016, 26, 1-4.	1.7	11
13	Development of manufacturing technology for ITER TF Coil Structure. Fusion Engineering and Design, 2016, 109-111, 1592-1597.	1.9	6
14	ICONE23-1713 WELDING JOINT DESIGN OF ITER TOROIDAL FIELD COIL STRUCTURE UNDER CRYOGENIC ENVIRONMENT. The Proceedings of the International Conference on Nuclear Engineering (ICONE), 2015, 2015.23, _ICONE23-1-_ICONE23-1.	0.0	0
15	Performance Test of Diamond-Like Carbon Films for Lubricating ITER Blanket Maintenance Equipment under GPa-Level High Contact Stress. Plasma and Fusion Research, 2007, 2, 052-052.	0.7	6