Shanfang Huang

List of Publications by Year
in descending order

41 papers	548 citations	687363 13 h-index	677142 22 g-index
41 all docs	41 docs citations	41 times ranked	258 citing authors

#	Article	IF	Citations
1	Study on discharge coefficient of perforated orifices as a new kind of flowmeter. Experimental Thermal and Fluid Science, 2013, 46, 74-83.	2.7	67
2	Heat pipe failure accident analysis in megawatt heat pipe cooled reactor. Annals of Nuclear Energy, 2020, 149, 107755.	1.8	54
3	Neutronic and thermal-mechanical coupling analyses in a solid-state reactor using Monte Carlo and finite element methods. Annals of Nuclear Energy, 2021, 151, 107923.	1.8	40
4	BEAVRS full core burnup calculation in hot full power condition by RMC code. Annals of Nuclear Energy, 2017, 101, 434-446.	1.8	35
5	Gas flow rate measurement in low-quality multiphase flows using Venturi and gamma ray. Experimental Thermal and Fluid Science, 2019, 100, 319-327.	2.7	35
6	Effect of inclination angle on the startup of a frozen sodium heat pipe. Applied Thermal Engineering, 2022, 201, 117625.	6.0	33
7	Study on flow pattern maps in hilly-terrain air–water–oil three-phase flows. Experimental Thermal and Fluid Science, 2013, 47, 158-171.	2.7	27
8	Equivalent water layer height (EWLH) measurement by a single-wire capacitance probe in gas–liquid flows. International Journal of Multiphase Flow, 2008, 34, 809-818.	3.4	23
9	RMC/CTF multiphysics solutions to VERA core physics benchmark problem 9. Annals of Nuclear Energy, 2019, 133, 837-852.	1.8	23
10	Transient heat pipe failure accident analysis of a megawatt heat pipe cooled reactor. Progress in Nuclear Energy, 2021, 140, 103904.	2.9	23
11	A new model for volume fraction measurements of horizontal high-pressure wet gas flow using gamma-based techniques. Experimental Thermal and Fluid Science, 2018, 96, 311-320.	2.7	22
12	Coupled neutronic, thermal-mechanical and heat pipe analysis of a heat pipe cooled reactor. Nuclear Engineering and Design, 2021, 384, 111473.	1.7	20
13	Heat Transfer Calculation on Plate-Type Fuel Assembly of High Flux Research Reactor. Science and Technology of Nuclear Installations, 2015, 2015, 1-13.	0.8	14
14	Improved adaptive variance reduction algorithm based on RMC code for deep penetration problems. Annals of Nuclear Energy, 2020, 137, 107113.	1.8	14
15	A flow rate measurement method for horizontal oil-gas-water three-phase flows based on Venturi meter, blind tee, and gamma-ray attenuation. Flow Measurement and Instrumentation, 2021, 80, 101965.	2.0	13
16	Neutronics and thermal-hydraulics coupling analysis in accelerator-driven subcritical system. Progress in Nuclear Energy, 2020, 122, 103235.	2.9	12
17	Coupled irradiation-thermal-mechanical analysis of the solid-state core in a heat pipe cooled reactor. Nuclear Engineering and Technology, 2022, 54, 2094-2106.	2.3	11
18	Local Void Fractions and Bubble Velocity in Vertical Air-Water Two-Phase Flows Measured by Needle-Contact Capacitance Probe. Science and Technology of Nuclear Installations, 2018, 2018, 1-14.	0.8	10

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19	Kinetic methods in Monte Carlo code RMC and its implementation to C5G7-TD benchmark. Annals of Nuclear Energy, 2021, 151, 107864.	1.8	9
20	Criticality benchmarking of ENDF/B-â…§.0 and JEFF-3.3 neutron data libraries with RMC code. Nuclear Engineering and Technology, 2020, 52, 1917-1925.	2.3	8
21	A method of extending subcritical heat transfer correlations to supercritical conditions. Nuclear Engineering and Design, 2014, 266, 186-193.	1.7	6
22	Superhistory-based differential operator method for generalized responses sensitivity calculations. Annals of Nuclear Energy, 2020, 140, 107291.	1.8	6
23	Calculating the k-Eigenvalue Sensitivity to Typical Geometric Perturbations with the Adjoint-Weighted Method in the Continuous-Energy Reactor Monte Carlo Code RMC. Nuclear Science and Engineering, 2019, 193, 1186-1218.	1.1	5
24	Improvement of sensitivity and uncertainty analysis capabilities of generalized response in Monte Carlo code RMC. Annals of Nuclear Energy, 2021, 154, 108099.	1.8	5
25	Convergence diagnostics for Monte Carlo fission source distributions using the Wasserstein distance measure. Nuclear Engineering and Design, 2022, 389, 111675.	1.7	5
26	An improved tracking method for particle transport Monte Carlo simulations. Journal of Computational Physics, 2021, 437, 110330.	3.8	4
27	Prediction of Flow and Temperature Distributions in a High Flux Research Reactor Using the Porous Media Approach. Science and Technology of Nuclear Installations, 2017, 2017, 1-13.	0.8	3
28	Experimental study on steady-state heat transfer characteristics of the Nozzle-atomized dispersed flow. Applied Thermal Engineering, 2018, 140, 686-695.	6.0	3
29	A hash mapping method using cell vectors in Monte Carlo code RMC. Annals of Nuclear Energy, 2021, 160, 108395.	1.8	3
30	A TRANSIENT AND STEADY-STATE NETWORK MODEL FORANNULAR-WICK HEAT PIPES IN CONTINUUM FLOW PATTERN. The Proceedings of the International Conference on Nuclear Engineering (ICONE), 2019, 2019.27, 1363.	0.0	3
31	Geometric sensitivity analysis of generalized response function with RMC code. Annals of Nuclear Energy, 2020, 149, 107824.	1.8	2
32	Improved generalized perturbation theory method for sensitivity analysis of generalized response function. Progress in Nuclear Energy, 2021, 134, 103643.	2.9	2
33	Neutronic and Thermal-Mechanical Coupling Schemes for Heat Pipe-Cooled Reactor Designs. Journal of Nuclear Engineering and Radiation Science, 2022, 8, .	0.4	2
34	Development of an improved direct kinetic simulation capability in RMC code. Annals of Nuclear Energy, 2022, 173, 109110.	1.8	2
35	CFD Analysis of the Passive Decay Heat Removal System of an LBE-Cooled Fast Reactor. Science and Technology of Nuclear Installations, 2018, 2018, 1-11.	0.8	1
36	Temperature perturbation method using on-the-fly treatment of the cross-sections in the resolved resonance region. Annals of Nuclear Energy, 2021, 159, 108329.	1.8	1

#	Article	IF	Citations
37	Single neutron tracking method for calculating the probability of survival based on the RMC code. Annals of Nuclear Energy, 2022, 165, 108763.	1.8	1
38	Research on global neighbor list method in Monte Carlo code RMC. Annals of Nuclear Energy, 2022, 167, 108861.	1.8	1
39	Optimization of spatial structure designs of control rod using Monte Carlo code RMC. Frontiers in Energy, 0, , 1.	2.3	0
40	INVERSE UNCERTAINTY QUANTIFICATION OF CTF PHYSICAL MODEL PARAMETERS USING BAYESIAN INFERENCE. The Proceedings of the International Conference on Nuclear Engineering (ICONE), 2019, 2019.27, 1435.	0.0	0
41	EXPLORATION OF TRAINING MODE FOR PHD STUDENTS OF NUCLEAR ENGINEERING. The Proceedings of the International Conference on Nuclear Engineering (ICONE), 2019, 2019.27, 1376.	0.0	0