Shuhei Nogami

List of Publications by Year
in descending order

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		201674	214800
97	2,484	27	47
papers	citations	h-index	g-index

97	97	97	1094
all docs	docs citations	times ranked	citing authors

#	Article	IF	Citations
1	Tungstenâ€“tantalum alloys for fusion reactor applications. Journal of Nuclear Materials, 2022, 566, 153740.	2.7	15
2	Irradiation damages of structural materials under different irradiation environments. Journal of Nuclear Materials, 2021, 543, 152503.	2.7	11
3	Mechanical properties of tungsten: Recent research on modified tungsten materials in Japan. Journal of Nuclear Materials, 2021, 543, 152506.	2.7	55
4	Thermal diffusivity of irradiated tungsten and tungsten-rhenium alloys. Journal of Nuclear Materials, 2021, 543, 152594.	2.7	13
5	Ductile to brittle transition temperature of advanced tungsten alloys for nuclear fusion applications deduced by miniaturized three-point bending tests. International Journal of Refractory Metals and Hard Materials, 2021, 95, 105464.	3.8	33
6	Submicron-thick yttria-stabilized zirconia coating as an advanced tritium permeation barrier. Nuclear Fusion, 2021, 61, 076015.	3.5	5
7	Neutron irradiation tolerance of potassium-doped and rhenium-alloyed tungsten. Journal of Nuclear Materials, 2021, 553, 153009.	2.7	21
8	Helium Effects on Tensile Properties of Powder Metallurgical-Processed Tungsten for Fusion Reactor Applications. Nuclear Materials and Energy, 2021, 29, 101076.	1.3	1
9	Development of small bulge fatigue testing technique using small diskâ€type specimen. Fatigue and Fracture of Engineering Materials and Structures, 2020, 43, 444-456.	3.4	3
10	Tungsten modified by potassium doping and rhenium addition for fusion reactor applications. Fusion Engineering and Design, 2020, 152, 111445.	1.9	33
11	Fatigue properties of ferritic/martensitic steel after neutron irradiation and helium implantation. Nuclear Materials and Energy, 2020, 24, 100764.	1.3	4
12	Laminated composites using potassium doped tungsten. Fusion Engineering and Design, 2020, 161, 111894.	1.9	1
13	Helium effects on recovery and recrystallization of powder metallurgically processed tungsten. Physica Scripta, 2020, T171, 014016.	2.5	8
14	Tensile and impact properties of tungsten-rhenium alloy for plasma-facing components in fusion reactor. Fusion Engineering and Design, 2019, 148, 111323.	1.9	27
15	Improvement of impact properties of tungsten by potassium doping. Fusion Engineering and Design, 2019, 140, 48-61.	1.9	24
16	Effects of helium on mechanical properties of tungsten for fusion applications. Nuclear Materials and Energy, 2018, 15, 154-157.	1.3	22
17	Recrystallization behavior of hot-rolled pure tungsten and its alloy plates during high-temperature annealing. Nuclear Materials and Energy, 2018, 15, 158-163.	1.3	71
18	Solid state diffusion bonding of doped tungsten alloys with different thermo-mechanical properties. Fusion Engineering and Design, 2018, 136, 76-81.	1.9	8

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19	Effect of neutron irradiation on rhenium cluster formation in tungsten and tungsten-rhenium alloys. Journal of Nuclear Materials, 2018, 507, 78-86.	2.7	47
20	Thermal properties of pure tungsten and its alloys for fusion applications. Fusion Engineering and Design, 2018, 132, 1-6.	1.9	64
21	Japanese activities of the R&D on silicon carbide composites in the broader approach period and beyond. Journal of Nuclear Materials, 2018, 511, 582-590.	2.7	6
22	A review of impact properties of tungsten materials. Fusion Engineering and Design, 2018, 135, 196-203.	1.9	30
23	Deformation of dissimilar-metals joint between F82H and 316L in impact tests after neutron irradiation. Fusion Engineering and Design, 2017, 124, 1063-1067.	1.9	6
24	Fatigue life prediction of ferritic/martensitic steels based on universal slope equations and tensile properties obtained using small specimen. Fusion Engineering and Design, 2017, 125, 330-336.	1.9	3
25	Fatigue Properties of SiC/SiC Composites Under Various Loading Modes. Fusion Science and Technology, 2017, , 1-6.	1.1	3
26	Degradation of tungsten monoblock divertor under cyclic high heat flux loading. Fusion Engineering and Design, 2017, 120, 49-60.	1.9	28
27	Feasibility of Utilizing Tungsten Rod for Fusion Reactor Divertor. Fusion Science and Technology, 2017, 72, 673-679.	1.1	17
28	High Temperature Fatigue Life Evaluation Using Small Specimen. Plasma and Fusion Research, 2017, 12, 1405022-1405022.	0.7	4
29	Analysis of the temperature and thermal stress in pure tungsten monoblock during heat loading and the influences of alloying and dispersion strengthening on these responses. Fusion Engineering and Design, 2016, 107, 44-50.	1.9	15
30	The effect of voids on the hardening of body-centered cubic Fe. Journal of Nuclear Materials, 2016, 471, 233-238.	2.7	15
31	Effect of self-ion irradiation on hardening and microstructure of tungsten. Nuclear Materials and Energy, 2016, 9, 430-435.	1.3	44
32	Influence of friction stir welding conditions on joinability of oxide dispersion strengthened steel / F82H ferritic/martensitic steel joint. Nuclear Materials and Energy, 2016, 9, 367-371.	1.3	9
33	Effect of microstructural anisotropy on the mechanical properties of K-doped tungsten rods for plasma facing components. Fusion Engineering and Design, 2016, 109-111, 1549-1553.	1.9	19
34	Neutron irradiation effects on the microstructural development of tungsten and tungsten alloys. Journal of Nuclear Materials, 2016, 471, 175-183.	2.7	151
35	Strain rate dependence of tensile properties of tungsten alloys for plasma-facing components in fusion reactors. Fusion Engineering and Design, 2016, 109-111, 1674-1677.	1.9	26
36	Plastic deformation behavior and bonding strength of an EBW joint between 9Cr-ODS and JLF-1 estimated by symmetric four-point bend tests combined with FEM analysis. Fusion Engineering and Design, 2016, 102, 88-93.	1.9	5

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37	Tensile and fatigue properties of potassium doped and rhenium containing tungsten rods for fusion reactor applications. Fusion Engineering and Design, 2016, 109-111, 1538-1542.	1.9	17
38	Evaluation of impacts of stress triaxiality on plastic deformability of RAFM steel using various types of tensile specimen. Fusion Engineering and Design, 2016, 109-111, 1631-1636.	1.9	7
39	Anisotropy in the Mechanical Properties of Potassium and Rhenium Doped Tungsten Alloy Plates for Fusion Reactor Applications. Fusion Science and Technology, 2015, 68, 690-693.	1.1	35
40	Effect of Grain Structure Anisotropy and Recrystallization on Tensile Properties of Swaged Tungsten Rod. Plasma and Fusion Research, 2015, 10, 1405073-1405073.	0.7	8
41	Weldability of 9Cr-ODS and JLF-1 Steels for Dissimilar Joining with Hot Isostatic Pressing and Electron Beam Welding. Plasma and Fusion Research, 2015, 10, 3405015-3405015.	0.7	10
42	Effects of temperature and strain rate on the tensile properties of potassium-doped tungsten. Journal of Nuclear Materials, 2015, 461, 357-364.	2.7	33
43	Effect of post-weld heat treatment and neutron irradiation on a dissimilar-metal joint between F82H steel and 316L stainless steel. Fusion Engineering and Design, 2015, 98-99, 1968-1972.	1.9	12
44	Overview on recent progress toward small specimen test technique. Fusion Engineering and Design, 2015, 98-99, 2089-2093.	1.9	17
45	Effect of He implantation on fracture behavior and microstructural evolution in F82H. Journal of Nuclear Materials, 2014, 455, 690-694.	2.7	9
46	Tensile properties of K-doped Wâ€“3%Re. Fusion Engineering and Design, 2014, 89, 1033-1036.	1.9	24
47	Effects of tool rotation speed on the mechanical properties and microstructure of friction stir welded ODS steel. Materials Science & Engineering A: Structural Materials: Properties, Microstructure and Processing, 2014, 595, 291-296.	5.6	38
48	Microstructural development of tungsten and tungstenâ€“rhenium alloys due to neutron irradiation in HFIR. Journal of Nuclear Materials, 2014, 455, 460-463.	2.7	118
49	Evaluation of irradiation hardening of ion-irradiated Vâ€“4Crâ€“4Ti and Vâ€“4Crâ€“4Tiâ€“0.15Y alloys by nanoindentation techniques. Journal of Nuclear Materials, 2014, 455, 440-444.	2.7	30
50	Neutron irradiation effects on tungsten materials. Fusion Engineering and Design, 2014, 89, 1568-1572.	1.9	167
51	Depth-dependent nanoindentation hardness of reduced-activation ferritic steels after MeV Fe-ion irradiation. Fusion Engineering and Design, 2014, 89, 1637-1641.	1.9	40
52	Evaluation of irradiation hardening of proton irradiated stainless steels by nanoindentation. Journal of Nuclear Materials, 2014, 446, 142-147.	2.7	70
53	Microstructure development of dispersion-strengthened tungsten due to neutron irradiation. Journal of Nuclear Materials, 2014, 449, 213-218.	2.7	34
54	Structural Optimization of the Blanket First Wall to Reduce Thermal Stress Using the Taguchi Method. Plasma and Fusion Research, 2014, 9, 1405143-1405143.	0.7	0

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55	Effect of helium on fatigue crack growth and life of reduced activation ferritic/martensitic steel. Journal of Nuclear Materials, 2013, 442, S43-S47.	2.7	6
56	Study on electron beam weld joints between pure vanadium and SUS316L stainless steel. Journal of Nuclear Materials, 2013, 442, S562-S566.	2.7	23
57	Effect of heat treatment on bend stress relaxation of pure tungsten. Fusion Engineering and Design, 2013, 88, 1735-1738.	1.9	5
58	Development of fatigue life evaluation method using small specimen. Journal of Nuclear Materials, 2013, 441, 125-132.	2.7	23
59	Tritium trapping in silicon carbide in contact with solid breeder under high flux isotope reactor irradiation. Journal of Nuclear Materials, 2013, 442, S497-S500.	2.7	4
60	Property change of advanced tungsten alloys due to neutron irradiation. Journal of Nuclear Materials, 2013, 442, S273-S276.	2.7	95
61	Neutron Irradiation Behavior of Tungsten. Materials Transactions, 2013, 54, 466-471.	1.2	96
62	Evaluation of Feasibility of Tungsten/Oxide Dispersion Strengthened Steel Bonding with Vanadium Insert. Materials Transactions, 2013, 54, 451-455.	1.2	7
63	Bend Stress Relaxation of Pure Tungsten. Plasma and Fusion Research, 2013, 8, 1405006-1405006.	0.7	2
64	Effects of Re Content and Fabrication Process on Microstructural Changes and Hardening in Neutron Irradiated Tungsten. Materials Transactions, 2012, 53, 2145-2150.	1.2	84
65	Study on Fatigue Life Evaluation Using Small Specimens for Testing Neutron-Irradiated Materials. Journal of Nuclear Science and Technology, 2011, 48, 60-64.	1.3	12
66	Design plan and requirement of test module and testing items in IFMIF. Fusion Engineering and Design, 2011, 86, 712-715.	1.9	6
67	Fatigue Crack Initiation in Proton-Irradiated Austenitic Stainless Steel. Journal of Nuclear Science and Technology, 2011, 48, 1265-1271.	1.3	3
68	Indentation Properties of Silicon Carbide after Neutron Irradiation and Helium Implantation. IOP Conference Series: Materials Science and Engineering, 2011, 18, 162007.	0.6	5
69	Microstructure Development in Neutron Irradiated Tungsten Alloys. Materials Transactions, 2011, 52, 1447-1451.	1.2	112
70	Effect of specimen shape on micro-crack growth behavior under fatigue in reduced activation ferritic/martensitic steel. Journal of Nuclear Materials, 2011, 417, 131-134.	2.7	10
71	Property change mechanism in tungsten under neutron irradiation in various reactors. Journal of Nuclear Materials, 2011, 417, 491-494.	2.7	99
72	Helium effects on the tensile property of 316FR stainless steel at 650 and 750 Â°C. Journal of Nuclear Materials, 2011, 417, 1030-1033.	2.7	3

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73	Study on stress relaxation behavior of silicon carbide by BSR method. Journal of Nuclear Materials, 2011, 417, 356-358.	2.7	8
74	Small specimen test technology and methodology of IFMIF/EVEDA and the further subjects. Journal of Nuclear Materials, 2011, 417, 1325-1330.	2.7	26
75	High-Temperature Helium Embrittlement of 316FR Steel. Journal of Nuclear Science and Technology, 2011, 48, 130-134.	1.3	16
76	Study on Fatigue Life Evaluation Using Small Specimens for Testing Neutron-Irradiated Materials. Journal of Nuclear Science and Technology, 2011, 48, 60-64.	1.3	2
77	OS2504 Evaluation of Micro-crack Initiation Behavior in Irradiated Stainless Steel. The Proceedings of the Materials and Mechanics Conference, 2011, 2011, _OS2504-1_-_OS2504-2_.	0.0	0
78	Effect of Specimen Shape on the Low Cycle Fatigue Life of Reduced Activation Ferritic/Martensitic Steel. Journal of Nuclear Science and Technology, 2010, 47, 47-52.	1.3	15
79	Fatigue Life Assessment Based on Crack Growth Behavior in Reduced Activation Ferritic/Martensitic Steel. Journal of Nuclear Science and Technology, 2010, 47, 457-461.	1.3	6
80	Fatigue Life Assessment Based on Crack Growth Behavior in Reduced Activation Ferritic/Martensitic Steel. Journal of Nuclear Science and Technology, 2010, 47, 457-461.	1.3	1
81	Effect of Specimen Shape on the Low Cycle Fatigue Life of Reduced Activation Ferritic/Martensitic Steel. Journal of Nuclear Science and Technology, 2010, 47, 47-52.	1.3	3
82	1111 Study on Prediction of Fatigue Crack Initiation in Irradiated Material using Electron Backscatter Diffraction. The Proceedings of the Materials and Mechanics Conference, 2010, 2010, 1012-1013.	0.0	0
83	1608 Current Status of Development of Small Specimen Test Technique of Fatigue for Evaluation of 14 MeV Neutron Irradiated Materials. The Proceedings of the Materials and Mechanics Conference, 2010, 2010, 242-243.	0.0	0
84	S0306-4-4 Study on Prediction of Fatigue Crack Initiation Life in Austenitic Stainless Steels. The Proceedings of the JSME Annual Meeting, 2010, 2010.1, 49-50.	0.0	0
85	Micro-crack growth behavior and life in high temperature low cycle fatigue of blade root and disc joint for turbines. International Journal of Pressure Vessels and Piping, 2009, 86, 622-627.	2.6	5
86	Compatibility between SiC and Li ceramics for solid breeding blanket system. Journal of Nuclear Materials, 2009, 386-388, 628-630.	2.7	9
87	Improvement of Surface Exfoliation Behavior by Helium-ion Bombardment of a Tungsten Alloy Fabricated by Mechanical Alloying. Journal of Nuclear Science and Technology, 2009, 46, 717-723.	1.3	12
88	Improvement of Surface Exfoliation Behavior by Helium-ion Bombardment of a Tungsten Alloy Fabricated by Mechanical Alloying. Journal of Nuclear Science and Technology, 2009, 46, 717-723.	1.3	2
89	Oxidation behavior of SiC/SiC composites for helium cooled solid breeder blanket. Fusion Engineering and Design, 2008, 83, 1490-1494.	1.9	17
90	Precipitation of Solid Transmutation Elements in Irradiated Tungsten Alloys. Materials Transactions, 2008, 49, 2259-2264.	1.2	62

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91	Microstructure Evaluation of Advanced W Alloy with Fine Grain Structure Superior to Helium Irradiation Property. Materia Japan, 2008, 47, 633-633.	0.1	0
92	Effects of Transmutation Elements on Neutron Irradiation Hardening of Tungsten. Materials Transactions, 2007, 48, 2399-2402.	1.2	120
93	Micro Crack Growth Behavior and Life in High Temperature Low Cycle Fatigue of Blade Root and Disc Joint for Turbines. Zairyo/Journal of the Society of Materials Science, Japan, 2007, 56, 150-156.	0.2	4
94	Effect of He pre-implantation and neutron irradiation on mechanical properties of SiC/SiC composite. Journal of Nuclear Materials, 2004, 329-333, 577-581.	2.7	11
95	Surface Morphology Changes in a SiC/SiC Composite as Caused by Simultaneous Triple-Ion-Beam Irradiation. Materials Transactions, 2001, 42, 171-175.	1.2	5
96	Helium-bubble formation behavior of SiCf/SiC composites after helium implantation. Journal of Nuclear Materials, 1999, 264, 355-358.	2.7	48
97	Effects of Helium Production and Displacement Damage on Microstructural Evolution and Mechanical Properties in Helium-Implanted Austenitic Stainless Steel and Ferritic/Martensitic Steel. Materials Science Forum, 0, 1024, 53-69.	0.3	1