

Å tefan ÄŒerba

List of Publications by Year in descending order

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Version: 2024-02-01

26
papers

98
citations

1478505

6
h-index

1372567

10
g-index

26
all docs

26
docs citations

26
times ranked

60
citing authors

#	ARTICLE	IF	CITATIONS
1	Core neutronics characterization of the GFR2400 Gas Cooled Fast Reactor. Progress in Nuclear Energy, 2015, 83, 460-481.	2.9	31
2	A neutronic feasibility study on a small LEU fueled reactor for space applications. Annals of Nuclear Energy, 2015, 77, 35-46.	1.8	14
3	Comparison of thermal scattering processing options for S($\hat{\pm}$, $\hat{\pm}^2$) cards in MCNP. Annals of Nuclear Energy, 2013, 55, 18-22.	1.8	8
4	On gas-cooled fast reactor designs " Nuclear data processing with sensitivity, uncertainty and similarity analyses. Progress in Nuclear Energy, 2020, 128, 103450.	2.9	8
5	Unmanned Radiation-Monitoring System. IEEE Transactions on Nuclear Science, 2020, 67, 636-643.	2.0	8
6	Optimization of the heterogeneous GFR 2400 control rod design. Progress in Nuclear Energy, 2017, 97, 170-181.	2.9	7
7	The Mini Labyrinth Benchmark for Radiation Protection and Shielding Analysis. IEEE Transactions on Nuclear Science, 2022, 69, 745-752.	2.0	5
8	Verification of spectral burn-up codes on 2D fuel assemblies of the GFR demonstrator ALLEGRO reactor. Nuclear Engineering and Design, 2014, 267, 148-153.	1.7	4
9	Cross section adjustment for fast reactor design: ATCROSS code and its application. Annals of Nuclear Energy, 2017, 110, 928-940.	1.8	4
10	RADIATION MONITORING SYSTEM USING UNMANNED AERIAL VEHICLES. Radiation Protection Dosimetry, 2019, 186, 337-341.	0.8	2
11	Multigroup cross section library for GFR2400. EPJ Web of Conferences, 2017, 146, 09031.	0.3	1
12	Criticality safety validation of the SCALE system. AIP Conference Proceedings, 2018, , .	0.4	1
13	The VVER-440 burnup credit computational benchmark used for the SCALE system qualification. AIP Conference Proceedings, 2018, , .	0.4	1
14	Reactivity response analysis of a movable reflector system for the GFR 2400 core. AIP Conference Proceedings, 2018, , .	0.4	1
15	Cross Section and Fission Yields Induced Uncertainty in the Water"Water Energetic Reactor-440 Burnup Calculation. Journal of Nuclear Engineering and Radiation Science, 2022, 8, .	0.4	1
16	Preliminary results of the STU mini labyrinth radiation shielding experiment. AIP Conference Proceedings, 2021, , .	0.4	1
17	The Mini Labyrinth " a Simple Benchmark for Radiation Protection and Shielding Analysis. EPJ Web of Conferences, 2021, 253, 07013.	0.3	1
18	Estimation of the dose rate in the preparation phase of the neutron emission rate measurement. AIP Conference Proceedings, 2018, , .	0.4	0

#	ARTICLE	IF	CITATIONS
19	Update of the SBJ_V2019T XS library for multi-group and continuous-energy MCNP calculations of VVER reactors. AIP Conference Proceedings, 2019, , .	0.4	0
20	The measurement of diffusion length of demi water by neutron activation of foils vs. Helium-3 neutron detector. AIP Conference Proceedings, 2019, , .	0.4	0
21	The simplified burnup benchmark based on the novovoronezh NPP fuel assembly. AIP Conference Proceedings, 2019, , .	0.4	0
22	Development and Processing of Thermal-Hydraulic Model for GFR 2400 Fast Reactor Design and NESTLE Coupled Transient Code. Journal of Nuclear Engineering and Radiation Science, 2022, 8, .	0.4	0
23	Implementation of engineering codes for WWER-440 core calculations. Journal of Electrical Engineering, 2019, 70, 52-57.	0.7	0
24	Comparison of neutronic parameters for developed GFR 2400 cross-section library in NESTLE code. AIP Conference Proceedings, 2021, , .	0.4	0
25	Development and calibration of the units for dosimetry monitoring system. AIP Conference Proceedings, 2021, , .	0.4	0
26	On the Taylor series solution of the reactor point kinetics equations. AIP Conference Proceedings, 2021, , .	0.4	0