

Yi Xie

List of Publications by Year in descending order

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Version: 2024-02-01

30
papers

349
citations

840776

11
h-index

839539

18
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30
all docs

30
docs citations

30
times ranked

234
citing authors

#	ARTICLE	IF	CITATIONS
1	Chloride-induced stress corrosion cracking of used nuclear fuel welded stainless steel canisters: A review. <i>Journal of Nuclear Materials</i> , 2015, 466, 85-93.	2.7	51
2	Characterization of stress corrosion cracks in Ni-based weld alloys 52, 52M and 152 grown in high-temperature water. <i>Materials Characterization</i> , 2016, 112, 87-97.	4.4	34
3	Characterization of U-Zr fuel with alloying additive Sb for immobilizing fission product lanthanides. <i>Journal of Nuclear Materials</i> , 2018, 498, 332-340.	2.7	29
4	Revealing 3D Morphological and Chemical Evolution Mechanisms of Metals in Molten Salt by Multimodal Microscopy. <i>ACS Applied Materials & Interfaces</i> , 2020, 12, 17321-17333.	8.0	20
5	Corrosion and deposition on the secondary circuit of steam generators. <i>Journal of Nuclear Science and Technology</i> , 2016, 53, 1455-1466.	1.3	19
6	Characterization of U-10Zr-2Sn-2Sb and U-10Zr-2Sn-2Sb-4Ln to assess Sn+Sb as a mixed additive system to bind lanthanides. <i>Journal of Nuclear Materials</i> , 2018, 510, 210-218.	2.7	19
7	Effects of flow, Si inhibition, and concurrent corrosion of dissimilar metals on the corrosion of aluminium in the environment following a loss-of-coolant accident. <i>Corrosion Science</i> , 2017, 128, 100-109.	6.6	16
8	Lanthanide migration and immobilization in metallic fuels. <i>Progress in Nuclear Energy</i> , 2018, 109, 233-238.	2.9	15
9	Out-of-pile and postirradiated examination of lanthanide and lanthanide-palladium interactions for metallic fuel. <i>Journal of Nuclear Materials</i> , 2021, 544, 152727.	2.7	15
10	Multi-state Markov modeling of pitting corrosion in stainless steel exposed to chloride-containing environment. <i>Reliability Engineering and System Safety</i> , 2018, 172, 239-248.	8.9	14
11	Assessment of Te as a U-Zr fuel additive to mitigate fuel-cladding chemical interactions. <i>Journal of Nuclear Materials</i> , 2019, 513, 175-184.	2.7	14
12	Microstructure Evolution of U-Zr System in A Thermal Cycling Neutron Diffraction Experiment: Extruded U-10Zr (wt. %). <i>Journal of Nuclear Materials</i> , 2021, 544, 152665.	2.7	12
13	XRD and SEM/EDS characterization of two quaternary fuel alloys (U-2.5Mo-2.5Ti-5.0Zr and U-2.5Mo-2.5Ti-5.0Zr-1.0Nb). <i>Journal of Nuclear Materials</i> , 2018, 498, 332-340.	4.4	11
14	Corrosion behaviour of stainless steel exposed to highly concentrated chloride solutions. <i>Corrosion Engineering Science and Technology</i> , 2017, 52, 283-293.	1.4	10
15	Diffusion behaviors between metallic fuel alloys with Pd addition and Fe. <i>Journal of Nuclear Materials</i> , 2019, 525, 111-124.	2.7	10
16	Thermodynamic stability studies of Ce-Sb compounds with Fe. <i>Journal of Nuclear Materials</i> , 2018, 499, 440-445.	2.7	9
17	Microstructural characterization of annealed U-20Pu-10Zr-3.86Pd and U-20Pu-10Zr-3.86Pd-4.3Ln. <i>Journal of Nuclear Materials</i> , 2019, 518, 287-297.	2.7	9
18	Interactions and immobilization of lanthanides with dopants in uranium-based metallic fuels. <i>Journal of Nuclear Materials</i> , 2020, 540, 152372.	2.7	9

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19	Diffusion behavior of lanthanide-additive compounds (Ce ₄ Sb ₃ , Ce ₂ Sb, and CeTe) against HT9 and Fe. <i>Materials Characterization</i> , 2019, 150, 107-117.	4.4	8
20	Experimental Investigation of FCCI Using Diffusion Couple Test Between UZr Fuel with Sb Additive and Cladding. <i>Nuclear Science and Engineering</i> , 2020, 194, 462-476.	1.1	5
21	Aluminum corrosion in reactor containment environment following a loss of coolant accident (LOCA): High-temperature flow loop tests. <i>Corrosion Science</i> , 2019, 151, 122-131.	6.6	3
22	Experimental assessment of antimony (Sb) in pure uranium for immobilizing fission product lanthanides. <i>Journal of Nuclear Materials</i> , 2020, 534, 152135.	2.7	3
23	Efficient computational search for lanthanide-binding additive dopants for advanced U-Zr based fuels. <i>Materialia</i> , 2020, 10, 100653.	2.7	3
24	Phase evolution of U-Zr system in a thermal cycling neutron diffraction experiment: as-cast U-35Zr and U-50Zr. <i>Journal of Nuclear Materials</i> , 2022, 564, 153681.	2.7	3
25	Aluminum alloy corrosion in boron-containing alkaline solutions. <i>Materials and Corrosion - Werkstoffe Und Korrosion</i> , 2019, 70, 810-819.	1.5	2
26	Microstructure study of U-35 wt.% Zr alloy after quick annealing at 650 Å°C. <i>Journal of Materials Research</i> , 2020, 35, 1095-1102.	2.6	2
27	Microstructure and diffusion behavior of uranium fuel with minor additives. <i>Journal of Nuclear Materials</i> , 2020, 535, 152200.	2.7	2
28	Solid-state phase transitions of two quaternary metallic fuel alloys (U-2.5Mo-2.5Ti-5.0Zr and) Tj ETQq0 0 0 rgBT /Overlock 10 Tf 50 382	2.7	2
29	Spent Fuel Interim Dry Storage System and Chloride-Induced Stress Corrosion Cracking. <i>Modern Nuclear Energy Analysis Methods</i> , 2018, , 177-236.	0.1	0
30	Chemical interactions between neodymium and advanced stainless steels. <i>Journal of Nuclear Materials</i> , 2022, 559, 153451.	2.7	0