

Roberto Bonifetto

List of Publications by Year in Descending Order

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Version: 2024-04-28

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The third column is the impact factor (IF) of the journal, and the fourth column is the number of citations of the article.

76
papers

616
citations

14
h-index

20
g-index

77
ext. papers

686
ext. citations

1.9
avg, IF

3.68
L-index

#	Paper	IF	Citations
76	AC losses in the second module of the ITER Central Solenoid. <i>IEEE Transactions on Applied Superconductivity</i> , 2022 , 1-1	1.8	4
75	Integrated design strategy for EU-DEMO first wall protection from plasma transients. <i>Fusion Engineering and Design</i> , 2022 , 177, 113067	1.7	5
74	DTT: A Challenging Framework for a Sound Superconducting Magnets Design. <i>IEEE Transactions on Applied Superconductivity</i> , 2022 , 32, 1-5	1.8	3
73	Improved Conceptual Design of the Beamline for the DTT Neutral Beam Injector. <i>IEEE Transactions on Plasma Science</i> , 2022 , 1-6	1.3	2
72	Analysis of the thermal-hydraulic effects of a plasma disruption on the DTT TF magnets. <i>IEEE Transactions on Applied Superconductivity</i> , 2022 , 1-1	1.8	3
71	Effect of local defects on HTS fusion magnets performance. <i>IEEE Transactions on Applied Superconductivity</i> , 2022 , 1-1	1.8	1
70	First ITER CS module test results. <i>Fusion Engineering and Design</i> , 2021 , 164, 112169	1.7	8
69	Identification of LOFA precursors in ITER superconducting magnet cryogenic cooling circuit. <i>Reliability Engineering and System Safety</i> , 2021 , 209, 107426	6.3	2
68	Conceptual design of a PAV-based tritium extractor for the WCLL breeding blanket of the EU DEMO: Effects of surface-limited vs. diffusion-limited modeling. <i>Fusion Engineering and Design</i> , 2021 , 167, 112363	1.7	0
67	Thermal Hydraulic Behavior of the First ITER CS Module. <i>IEEE Transactions on Applied Superconductivity</i> , 2021 , 31, 1-5	1.8	1
66	AC Losses in the First ITER CS Module Tests: Experimental Results and Comparison to Analytical Models. <i>IEEE Transactions on Applied Superconductivity</i> , 2021 , 31, 1-5	1.8	7
65	Development of the H4C Model of Quench Propagation in the ENEA HTS Cable-In-Conduit Conductor. <i>IEEE Transactions on Applied Superconductivity</i> , 2021 , 31, 1-5	1.8	3
64	Metamodeling and On-Line Clustering for Loss-of-Flow Accident Precursors Identification in a Superconducting Magnet Cryogenic Cooling Circuit. <i>Energies</i> , 2021 , 14, 5552	3.1	0
63	Thermal-hydraulic analysis of the DTT CS and PF pulsed coil performance during AC operation. <i>Fusion Engineering and Design</i> , 2021 , 173, 112836	1.7	1
62	Integrated deterministic and probabilistic safety assessment of a superconducting magnet cryogenic cooling circuit for nuclear fusion applications. <i>Reliability Engineering and System Safety</i> , 2020 , 201, 106945	6.3	5
61	Thermal-Hydraulic Analysis of the DTT Toroidal Field Magnets in DC Operation. <i>IEEE Transactions on Applied Superconductivity</i> , 2020 , 30, 1-5	1.8	6
60	Advance in the conceptual design of the European DEMO magnet system. <i>Superconductor Science and Technology</i> , 2020 , 33, 044013	3.1	22

59	Modeling Quench Propagation in the ENEA HTS Cable-In-Conduit Conductor. <i>IEEE Transactions on Applied Superconductivity</i> , 2020 , 30, 1-7	1.8	11
58	Identification of the Postulated Initiating Events of Accidents Occurring in a Toroidal Field Magnet of the EU DEMO. <i>Fusion Science and Technology</i> , 2019 , 75, 412-421	1.1	3
57	. <i>IEEE Transactions on Applied Superconductivity</i> , 2019 , 29, 1-7	1.8	4
56	. <i>IEEE Transactions on Applied Superconductivity</i> , 2019 , 29, 1-5	1.8	6
55	Analysis of a Protected Loss of Flow Accident (LOFA) in the ITER TF Coil Cooling Circuit. <i>IEEE Transactions on Applied Superconductivity</i> , 2018 , 28, 1-9	1.8	4
54	Performance Analysis of the NbTi PF Coils for the EU DEMO Fusion Reactor. <i>IEEE Transactions on Applied Superconductivity</i> , 2018 , 28, 1-5	1.8	4
53	Mechanical Analysis of the ENEA TF Coil Proposal for the EU DEMO Fusion Reactor. <i>IEEE Transactions on Applied Superconductivity</i> , 2018 , 28, 1-5	1.8	2
52	Prediction, experimental results and analysis of the ITER TF insert coil quench propagation tests, using the 4C code. <i>Superconductor Science and Technology</i> , 2018 , 31, 035004	3.1	11
51	Full-core coupled neutronic/thermal-hydraulic modelling of the EBR-II SHRT-45R transient. <i>International Journal of Energy Research</i> , 2018 , 42, 134-150	4.5	4
50	ThermalHydraulic Test and Analysis of the ENEA TF Conductor Sample for the EU DEMO Fusion Reactor. <i>IEEE Transactions on Applied Superconductivity</i> , 2018 , 28, 1-9	1.8	8
49	Progress in the design of the superconducting magnets for the EU DEMO. <i>Fusion Engineering and Design</i> , 2018 , 136, 1597-1604	1.7	54
48	Analysis of the cooldown of the ITER central solenoid model coil and insert coil. <i>Superconductor Science and Technology</i> , 2017 , 30, 015015	3.1	10
47	Analysis of AC Losses in the ITER Central Solenoid Insert Coil. <i>IEEE Transactions on Applied Superconductivity</i> , 2017 , 27, 1-5	1.8	14
46	DTT device: Conceptual design of the superconducting magnet system. <i>Fusion Engineering and Design</i> , 2017 , 122, 299-312	1.7	12
45	Analysis of Quench Propagation in the ITER Central Solenoid Insert (CSI) Coil. <i>IEEE Transactions on Applied Superconductivity</i> , 2017 , 27, 1-8	1.8	4
44	Analysis of the ITER central solenoid insert (CSI) coil stability tests. <i>Cryogenics</i> , 2017 , 85, 8-14	1.8	2
43	Analysis of the DC performance of the ITER CSI coil using the 4C code. <i>Fusion Engineering and Design</i> , 2017 , 124, 159-162	1.7	3
42	Performance analysis of a graded winding pack design for the EU DEMO TF coil in normal and off-normal conditions. <i>Fusion Engineering and Design</i> , 2017 , 124, 45-48	1.7	14

41	Characterization of the ITER CS conductor and projection to the ITER CS performance. <i>Fusion Engineering and Design</i> , 2017 , 124, 1-5	1.7	11
40	Quench Propagation in a TF Coil of the EU DEMO. <i>Fusion Science and Technology</i> , 2017 , 1-10	1.1	4
39	ITER Central Solenoid Insert Test Results. <i>IEEE Transactions on Applied Superconductivity</i> , 2016 , 1-1	1.8	22
38	Thermal-Hydraulic Modeling of a Novel HTS CICC for Nuclear Fusion Applications. <i>IEEE Transactions on Applied Superconductivity</i> , 2016 , 26, 1-7	1.8	8
37	Analysis of AC Losses in the EAST Superconducting Magnets Using the 4C Code. <i>IEEE Transactions on Applied Superconductivity</i> , 2016 , 26, 1-5	1.8	2
36	. <i>IEEE Transactions on Applied Superconductivity</i> , 2016 , 26, 1-6	1.8	19
35	Design and optimization of Artificial Neural Networks for the modelling of superconducting magnets operation in tokamak fusion reactors. <i>Journal of Computational Physics</i> , 2016 , 321, 476-491	4.1	6
34	Analyses of Low- and High-Margin Quench Propagation in the European DEMO TF Coil Winding Pack. <i>IEEE Transactions on Plasma Science</i> , 2016 , 44, 1564-1570	1.3	6
33	3D thermal-hydraulic analysis of two irregular field joints for the ITER vacuum vessel. <i>Fusion Engineering and Design</i> , 2015 , 98-99, 1605-1609	1.7	2
32	Incorporating Artificial Neural Networks in the dynamic thermal-hydraulic model of a controlled cryogenic circuit. <i>Cryogenics</i> , 2015 , 70, 9-20	1.8	5
31	4C code analysis of high-margin quench propagation in a DEMO TF coil 2015 ,		3
30	Artificial Neural Network Model for the Thermal-Hydraulic Response of a TF Superconducting Magnet in ITER. <i>Fusion Science and Technology</i> , 2015 , 68, 336-340	1.1	5
29	Thermal-hydraulic analysis of transients in the HELIOS loop including a CICC section representative of the JT-60SA Central Solenoid. <i>IOP Conference Series: Materials Science and Engineering</i> , 2015 , 101, 012147	0.4	
28	Artificial Neural Networks: a viable tool to design heat load smoothing strategies for the ITER Toroidal Field coils. <i>IOP Conference Series: Materials Science and Engineering</i> , 2015 , 101, 012149	0.4	
27	Multiscale Hydraulic Modeling of the ITER TF He Inlets During Nominal and Off-Normal Operation. <i>IEEE Transactions on Applied Superconductivity</i> , 2015 , 25, 1-5	1.8	
26	4C modeling of the supercritical helium loop HELIOS in isobaric configuration. <i>Cryogenics</i> , 2014 , 64, 51-62.8		2
25	Analysis of the Effects of the Nuclear Heat Load on the ITER TF Magnets Temperature Margin. <i>IEEE Transactions on Applied Superconductivity</i> , 2014 , 24, 1-4	1.8	8
24	Application of the 4C code to the thermal-hydraulic analysis of the CS superconducting magnets in EAST. <i>Cryogenics</i> , 2014 , 63, 255-262	1.8	6

23	Artificial Neural Network (ANN) modeling of the pulsed heat load during ITER CS magnet operation. <i>Cryogenics</i> , 2014 , 63, 231-240	1.8	13
22	3D thermal-hydraulic analysis of an ITER vacuum vessel regular Field Joint. <i>Fusion Engineering and Design</i> , 2014 , 89, 1848-1853	1.7	2
21	Effects of RANS-type Turbulence Models on the Convective Heat Loss Computed by CFD in the Solar Two Power Tower. <i>Energy Procedia</i> , 2014 , 49, 569-578	2.3	3
20	CFD analysis of the ITER first wall 06 panel. Part II: Thermal-hydraulics. <i>Fusion Engineering and Design</i> , 2014 , 89, 431-441	1.7	3
19	Evaluation of the neutron activation of JET in-vessel components following DT irradiation. <i>Fusion Engineering and Design</i> , 2014 , 89, 2071-2075	1.7	3
18	Verification of the predictive capabilities of the 4C code cryogenic circuit model 2014 ,		16
17	CFD analysis of the ITER first wall 06 panel. Part I: Model set-up and flow distribution. <i>Fusion Engineering and Design</i> , 2014 , 89, 442-455	1.7	
16	A full-core coupled neutronic/thermal-hydraulic code for the modeling of lead-cooled nuclear fast reactors. <i>Nuclear Engineering and Design</i> , 2013 , 261, 85-94	1.8	18
15	Modeling of pulsed heat load in a cryogenic SHe loop using Artificial Neural Networks. <i>Cryogenics</i> , 2013 , 57, 173-180	1.8	9
14	CFD analysis of a regular sector of the ITER vacuum vessel. Part I: Flow distribution and pressure drop. <i>Fusion Engineering and Design</i> , 2013 , 88, 3272-3279	1.7	9
13	4C code analysis of thermal-hydraulic transients in the KSTAR PF1 superconducting coil. <i>Cryogenics</i> , 2013 , 53, 37-44	1.8	24
12	Validation of the 4C code against data from the HELIOS loop at CEA Grenoble. <i>Cryogenics</i> , 2013 , 53, 25-308		16
11	Mitigation of the Temperature Margin Reduction due to the Nuclear Radiation on the ITER TF Coils. <i>IEEE Transactions on Applied Superconductivity</i> , 2013 , 23, 4201305-4201305	1.8	13
10	4C modeling of pulsed-load smoothing in the HELIOS facility using a controlled bypass valve. <i>Cryogenics</i> , 2013 , 57, 31-44	1.8	11
9	Thermal-Hydraulic Simulation of 80 kA Safety Discharge in the ITER Toroidal Field Model Coil (TFMC) Using the 4C Code. <i>IEEE Transactions on Plasma Science</i> , 2012 , 40, 782-787	1.3	10
8	4C Code Simulation and Benchmark of ITER TF Magnet Cool-Down From 300 K to 80 K. <i>IEEE Transactions on Applied Superconductivity</i> , 2012 , 22, 4902604-4902604	1.8	11
7	Parametric Analysis of the ITER TF Fast Discharge Using the 4C Code. <i>IEEE Transactions on Applied Superconductivity</i> , 2012 , 22, 4704104-4704104	1.8	21
6	Progress in Multi-Physics Modeling of Innovative Lead-Cooled Fast Reactors. <i>Fusion Science and Technology</i> , 2012 , 61, 293-297	1.1	1

5	Modeling of W7-X superconducting coil cool-down using the 4C code. <i>Fusion Engineering and Design</i> , 2011 , 86, 1549-1552	1.7	24
4	Computation of JT-60SA TF coil temperature margin using the 4C code. <i>Fusion Engineering and Design</i> , 2011 , 86, 1493-1496	1.7	17
3	Thermal-hydraulic simulation of 80 kA safety discharge in the ITER Toroidal Field Model Coil (TFMC) using the 4C code 2011 ,		1
2	Validation of the 4C Thermal-Hydraulic Code Against 25 kA Safety Discharge in the ITER Toroidal Field Model Coil (TFMC). <i>IEEE Transactions on Applied Superconductivity</i> , 2011 , 21, 1948-1952	1.8	30
1	Analysis of Quench Propagation in the ITER Poloidal Field Conductor Insert (PFCI). <i>IEEE Transactions on Applied Superconductivity</i> , 2010 , 20, 491-494	1.8	9