Wei Liu

List of Publications by Year
in descending order

18 papers	99 citations	1478505 6 h-index	1474206 9 g-index
18 all docs	18 docs citations	18 times ranked	22 citing authors

#	Article	IF	Citations
1	The visual experiment of boiling crisis triggering process during subcooled flow boiling. International Journal of Thermal Sciences, 2022, 172, 107347.	4.9	6
2	Numerical investigation of the critical heat flux in a 5ÂÃ— 5 rod bundle with multi-grid. Nuclear Engineering and Technology, 2022, 54, 1914-1928.	2.3	4
3	Existing DNB-type CHF mechanistic models and relations with visualized experiments in forced convective flow boiling: A review. Progress in Nuclear Energy, 2022, 148, 104225.	2.9	3
4	A phenomenological investigation on near-wall bubble behavior close to CHF in flow boiling. International Journal of Heat and Mass Transfer, 2022, 189, 122732.	4.8	5
5	Development and assessment of a new CHF mechanistic model for subcooled and low quality flow boiling. International Journal of Heat and Mass Transfer, 2021, 165, 120641.	4.8	5
6	Applicability research of round tube CHF mechanistic model in rod bundle channel. Nuclear Engineering and Technology, 2021, 53, 439-445.	2.3	2
7	Analytical investigation on rod bundle CHF-regime criterion based on dimensionless groups. International Journal of Thermal Sciences, 2021, 159, 106571.	4.9	3
8	Numerical investigation of the CHF in a vertical round tube and a single rod channel based on the eulerian two-fluid model. Progress in Nuclear Energy, 2021, 135, 103699.	2.9	7
9	Investigation on Rod Bundle CHF Mechanistic Model for DNB and DO Prediction Under Wide Parameter Range. Frontiers in Energy Research, 2021, 9, .	2.3	0
10	Visualization of spacer grid effect on bubble behavior and CHF in a single-rod channel. Nuclear Engineering and Design, 2021, 382, 111376.	1.7	5
11	Visual experimental study on bubble characteristics near the heating wall in subcooled flow boiling. Progress in Nuclear Energy, 2021, 140, 103898.	2.9	6
12	A mechanistic bubble crowding model for predicting critical heat flux in subchannels of a bundle. Annals of Nuclear Energy, 2020, 137, 107085.	1.8	13
13	Development and assessment of a new rod-bundle CHF correlation for China fuel assemblies. Annals of Nuclear Energy, 2020, 138, 107175.	1.8	9
14	A phenomenological CHF model for mixing-vane spacers in a subchannel of a rod bundle. Annals of Nuclear Energy, 2020, 142, 107445.	1.8	4
15	Application of the Improved Spacer Grid Model in Subchannel Analysis Code. Nuclear Technology, 2019, 205, 352-363.	1.2	4
16	The Study of Critical Heat Flux in Upflow Boiling Vertical Round Tube under High Pressure. Science and Technology of Nuclear Installations, 2019, 2019, 1-14.	0.8	2
17	Development and Application of a New High-Efficiency Sparse Linear System Solver in the Thermal-Hydraulic System Analysis Code. Science and Technology of Nuclear Installations, 2017, 2017, 1-10.	0.8	4
18	Ultrahigh CHF Prediction for Subcooled Flow Boiling Based on Homogenous Nucleation Mechanism. Journal of Heat Transfer, 2005, 127, 149-158.	2.1	17