Wei Li

List of Publications by Year
in descending order

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18	287	933447 10	940533 16
papers	citations	h-index	g-index

18	18	18	243
all docs	docs citations	times ranked	citing authors

#	Article	IF	Citations
1	Multiphysics phase-field modeling of quasi-static cracking in urania ceramic nuclear fuel. Ceramics International, 2021, 47, 793-810.	4.8	27
2	Application of Kriging and Variational Bayesian Monte Carlo method for improved prediction of doped UO2 fission gas release. Annals of Nuclear Energy, 2021, 153, 108046.	1.8	15
3	Finite element analysis of the SiC/SiC composite clad deformation in the presence of spacer grids. Annals of Nuclear Energy, 2020, 137, 107114.	1.8	1
4	Implications of SiC irradiation creep and annealing to UN-SiC fuel rod behavior. Journal of Nuclear Materials, 2020, 542, 152479.	2.7	8
5	Investigation on pressure fluctuation during parallel channel switching in fire-fighting. Proceedings of the Institution of Mechanical Engineers, Part C: Journal of Mechanical Engineering Science, 2020, 234, 2376-2385.	2.1	0
6	Multidimensional multiphysics modeling of TRISO particle fuel with SiC/ZrC coating using modified fission gas release model. Annals of Nuclear Energy, 2020, 145, 107599.	1.8	16
7	Innovative accident tolerant fuel concept enabled through direct manufacturing technology. Applied Energy, 2020, 264, 114742.	10.1	4
8	Modeling Axial Relocation of Fragmented Fuel During Loss of Coolant Conditions by Using ABAQUS. , 2020, , .		1
9	U3Si2-SiC fuel performance analysis in BISON during normal operation. Annals of Nuclear Energy, 2019, 132, 34-45.	1.8	26
10	ABAQUS analysis of the SiC cladding fuel rod behavior under PWR normal operation conditions. Journal of Nuclear Materials, 2019, 515, 14-27.	2.7	25
11	An investigation of numerical performance enhancement of RELAP5: Numerical stability, higher resolution and an alternative constitutive relation. Nuclear Engineering and Design, 2018, 328, 309-320.	1.7	3
12	Numerical research on water hammer phenomenon of parallel pump-valve system by coupling FLUENT with RELAP5. Annals of Nuclear Energy, 2017, 109, 318-326.	1.8	23
13	Preliminary safety analysis of the PWR with accident-tolerant fuels during severe accident conditions. Annals of Nuclear Energy, 2015, 80, 1-13.	1.8	42
14	Analysis of accidental loss of pool coolant due to leakage in a PWR SFP. Annals of Nuclear Energy, 2015, 77, 65-73.	1.8	10
15	Preliminary study of coupling CFD code FLUENT and system code RELAP5. Annals of Nuclear Energy, 2014, 73, 96-107.	1.8	42
16	Analysis of PWR RPV lower head SBLOCA scenarios with the failure of high-pressure injection system using MAAP5. Progress in Nuclear Energy, 2014, 77, 48-64.	2.9	4
17	Analysis of the loss of pool cooling accident in a PWR spent fuel pool with MAAP5. Annals of Nuclear Energy, 2014, 72, 198-213.	1.8	25
18	Preliminary thermal-hydraulic and safety analysis of China DFLL-TBM system. Fusion Engineering and Design, 2013, 88, 286-294.	1.9	15