

Andreas Kirschner

List of Publications by Year in descending order

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| # | ARTICLE | IF | CITATIONS |
|----|---|-----|-----------|
| 1 | Latest results of Eurofusion plasma-facing components research in the areas of power loading, material erosion and fuel retention. Nuclear Fusion, 2022, 62, 042013. | 1.6 | 11 |
| 2 | Operating a full tungsten actively cooled tokamak: overview of WEST first phase of operation. Nuclear Fusion, 2022, 62, 042007. | 1.6 | 39 |
| 3 | Experimental confirmation of efficient island divertor operation and successful neoclassical transport optimization in Wendelstein 7-X. Nuclear Fusion, 2022, 62, 042022. | 1.6 | 24 |
| 4 | Beryllium erosion and redeposition in ITER H, He and D ⁺ discharges. Nuclear Fusion, 2022, 62, 036011. | 1.6 | 13 |
| 5 | Predictive 3D modelling of erosion and deposition in ITER with ERO2.0: from beryllium main wall, tungsten divertor to full-tungsten device. Physica Scripta, 2022, 97, 014001. | 1.2 | 2 |
| 6 | Simulation Analysis of the Carbon Deposition Profile on Directional Material Probes in the Large Helical Device Using the ERO2.0 Code. Plasma and Fusion Research, 2022, 17, 2403010-2403010. | 0.3 | 1 |
| 7 | Simulation of Impurity Transport and Deposition in the Closed Helical Divertor in the Large Helical Device. Plasma and Fusion Research, 2021, 16, 2403004-2403004. | 0.3 | 2 |
| 8 | A sensitivity analysis of numerical predictions for beryllium erosion and migration in ITER. Nuclear Materials and Energy, 2021, 26, 100904. | 0.6 | 9 |
| 9 | Symmetries of ¹³ C tracer deposition in EAST D and He plasmas investigated on the sub-mm to 100 mm scale by deuterium nuclear reaction analysis. Fusion Engineering and Design, 2021, 166, 112292. | 1.0 | 1 |
| 10 | The impact of surface morphology on the erosion of metallic surfaces – Modelling with the 3D Monte-Carlo code ERO2.0. Nuclear Materials and Energy, 2021, 27, 100987. | 0.6 | 9 |
| 11 | Interpretative modeling of impurity transport and tungsten sources in WEST boundary plasma. Nuclear Fusion, 2021, 61, 126015. | 1.6 | 4 |
| 12 | Effectiveness of local methane and hydrogen injection into the scrape-off layer of W7-X by means of the multi-purpose manipulator. Fusion Engineering and Design, 2021, 173, 112786. | 1.0 | 1 |
| 13 | Plasma-wall interaction studies in W7-X: main results from the recent divertor operations. Physica Scripta, 2021, 96, 124059. | 1.2 | 10 |
| 14 | ¹³ C tracer deposition in EAST D and He plasmas investigated by high-throughput deuterium nuclear reaction analysis mapping. Nuclear Materials and Energy, 2020, 25, 100805. | 0.6 | 7 |
| 15 | Boron transport simulation using the ERO2.0 code for real-time wall conditioning in the large helical device. Nuclear Materials and Energy, 2020, 25, 100853. | 0.6 | 4 |
| 16 | Erosion and screening of tungsten during inter/intra-ELM periods in the JET-ILW divertor. Nuclear Materials and Energy, 2020, 25, 100859. | 0.6 | 7 |
| 17 | ERO2.0 modelling of the effects of surface roughness on molybdenum erosion and redeposition in the PSI-2 linear plasma device. Physica Scripta, 2020, T171, 014057. | 1.2 | 19 |
| 18 | First Monte-Carlo modelling of global beryllium migration in ITER using ERO2.0. Contributions To Plasma Physics, 2020, 60, e201900149. | 0.5 | 17 |

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| 19 | First efforts in numerical modeling of tungsten migration in WEST with SolEdge2D-EIRENE and ERO2.0. Physica Scripta, 2020, T171, 014013. | 1.2 | 16 |
| 20 | Fuel inventory and impurity deposition in castellated tungsten tiles in KSTAR: experiment and modelling. Physica Scripta, 2020, T171, 014049. | 1.2 | 2 |
| 21 | First divertor physics studies in Wendelstein 7-X. Nuclear Fusion, 2019, 59, 096014. | 1.6 | 34 |
| 22 | Overview of first Wendelstein 7-X high-performance operation. Nuclear Fusion, 2019, 59, 112004. | 1.6 | 165 |
| 23 | Erosion, screening, and migration of tungsten in the JET divertor. Nuclear Fusion, 2019, 59, 096035. | 1.6 | 60 |
| 24 | Modelling of tungsten erosion and deposition in the divertor of JET-ILW in comparison to experimental findings. Nuclear Materials and Energy, 2019, 18, 239-244. | 0.6 | 24 |
| 25 | Beryllium global erosion and deposition at JET-ILW simulated with ERO2.0. Nuclear Materials and Energy, 2019, 18, 331-338. | 0.6 | 36 |
| 26 | Improved ERO modelling of beryllium erosion at ITER upper first wall panel using JET-ILW and PISCES-B experience. Nuclear Materials and Energy, 2019, 19, 510-515. | 0.6 | 15 |
| 27 | Physics affecting heavy impurity migration in tokamaks: Benchmarking test-ion code ASCOT against TEXTOR tracer experiment. Nuclear Materials and Energy, 2019, 19, 307-315. | 0.6 | 1 |
| 28 | Surface roughness effect on Mo physical sputtering and re-deposition in the linear plasma device PSI-2 predicted by ERO2.0. Nuclear Materials and Energy, 2019, 19, 13-18. | 0.6 | 27 |
| 29 | Determination of tungsten sources in the JET-ILW divertor by spectroscopic imaging in the presence of a strong plasma continuum. Nuclear Materials and Energy, 2019, 18, 118-124. | 0.6 | 16 |
| 30 | ERO modeling and sensitivity analysis of locally enhanced beryllium erosion by magnetically connected antennas. Nuclear Fusion, 2018, 58, 016046. | 1.6 | 9 |
| 31 | Modelling of plasma-wall interaction and impurity transport in fusion devices and prompt deposition of tungsten as application. Plasma Physics and Controlled Fusion, 2018, 60, 014041. | 0.9 | 31 |
| 32 | Review on global migration, fuel retention and modelling after TEXTOR decommission. Nuclear Materials and Energy, 2018, 17, 83-112. | 0.6 | 9 |
| 33 | Surface roughness effects on plasma near a divertor plate and local impact angle. Nuclear Materials and Energy, 2017, 12, 313-317. | 0.6 | 9 |
| 34 | Modelling of deposition and erosion of injected WF6 and MoF6 in TEXTOR. Nuclear Materials and Energy, 2017, 12, 564-568. | 0.6 | 4 |
| 35 | Whole-machine material migration studies in the TEXTOR tokamak with molybdenum. Nuclear Materials and Energy, 2017, 12, 518-523. | 0.6 | 6 |
| 36 | Advances in understanding of high-Z material erosion and re-deposition in low-Z wall environment in DIII-D. Nuclear Fusion, 2017, 57, 056016. | 1.6 | 16 |

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| 37 | High-Z material erosion and its control in DIII-D carbon divertor. Nuclear Materials and Energy, 2017, 12, 247-252. | 0.6 | 4 |
| 38 | Fuel inventory and deposition in castellated structures in JET-ILW. Nuclear Fusion, 2017, 57, 066027. | 1.6 | 25 |
| 39 | ERO modelling of tungsten erosion and re-deposition in EAST L mode discharges. Physics of Plasmas, 2017, 24, 092512. | 0.7 | 20 |
| 40 | Quartz micro-balance results of pulse-resolved erosion/deposition in the JET-ILW divertor. Nuclear Materials and Energy, 2017, 12, 478-482. | 0.6 | 6 |
| 41 | Overview of wall probes for erosion and deposition studies in the TEXTOR tokamak. Matter and Radiation at Extremes, 2017, 2, 87-104. | 1.5 | 23 |
| 42 | ERO modelling of tungsten erosion in the linear plasma device PSI-2. Nuclear Materials and Energy, 2017, 12, 253-260. | 0.6 | 21 |
| 43 | ERO modeling of beryllium erosion by helium plasma in experiments at PISCES-B. Nuclear Materials and Energy, 2017, 12, 1157-1162. | 0.6 | 7 |
| 44 | ERO modeling of Cr sputtering in the linear plasma device PSI-2. Physica Scripta, 2017, T170, 014051. | 1.2 | 4 |
| 45 | Experimental data on low energy electron impact ionisation of W. Physica Scripta, 2017, T170, 014075. | 1.2 | 1 |
| 46 | Numerical and analytic study of rough surface morphology on the angular distribution of eroded impurity. Contributions To Plasma Physics, 2017, 57, 329-335. | 0.5 | 7 |
| 47 | An analytical expression for ion velocities at the wall including the sheath electric field and surface biasing for erosion modeling at JET ILW. Nuclear Materials and Energy, 2017, 12, 341-345. | 0.6 | 10 |
| 48 | First ERO2.0 modeling of Be erosion and non-local transport in JET ITER-like wall. Physica Scripta, 2017, T170, 014018. | 1.2 | 27 |
| 49 | Plasma-wall interactions in the presence of plasma fluctuations—interpretation of line emission from sputtered tungsten in PSI-2. Physica Scripta, 2017, T170, 014039. | 1.2 | 3 |
| 50 | Plasma—wall interaction studies within the EUROfusion consortium: progress on plasma-facing components development and qualification. Nuclear Fusion, 2017, 57, 116041. | 1.6 | 75 |
| 51 | Experimental estimation of tungsten impurity sputtering due to Type I ELMs in JET-ITER-like wall using pedestal electron cyclotron emission and target Langmuir probe measurements. Physica Scripta, 2016, T167, 014005. | 1.2 | 31 |
| 52 | Three-dimensional modeling of plasma edge transport and divertor fluxes during application of resonant magnetic perturbations on ITER. Nuclear Fusion, 2016, 56, 066008. | 1.6 | 34 |
| 53 | An Analytical Expression for the Electric Field and Particle Tracing in Modelling of Be Erosion Experiments at the JET ITER—like Wall. Contributions To Plasma Physics, 2016, 56, 640-645. | 0.5 | 26 |
| 54 | Improved ERO modelling for spectroscopy of physically and chemically assisted eroded beryllium from the JET-ILW. Nuclear Materials and Energy, 2016, 9, 604-609. | 0.6 | 17 |

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| 55 | Modelling of Impurity Transport and Plasma-Wall Interaction in Fusion Devices with the ERO Code: Basics of the Code and Examples of Application. Contributions To Plasma Physics, 2016, 56, 622-627. | 0.5 | 21 |
| 56 | Deposition in the inner and outer corners of the JET divertor with carbon wall and metallic ITER-like wall. Physica Scripta, 2016, T167, 014052. | 1.2 | 14 |
| 57 | Estimates of RF-induced erosion at antenna-connected beryllium plasma-facing components in JET. Physica Scripta, 2016, T167, 014035. | 1.2 | 14 |
| 58 | Simulation of gross and net erosion of high-Z materials in the DIII-D divertor. Nuclear Fusion, 2016, 56, 016021. | 1.6 | 41 |
| 59 | Local migration studies of high-Zmetals in the TEXTOR tokamak. Physica Scripta, 2016, T167, 014058. | 1.2 | 9 |
| 60 | Modelling of the material transport and layer formation in the divertor of JET: Comparison of ITER-like wall with full carbon wall conditions. Journal of Nuclear Materials, 2015, 463, 116-122. | 1.3 | 26 |
| 61 | Analysis of rotating collectors from the private region of JET with carbon wall and metallic ITER-like wall. Journal of Nuclear Materials, 2015, 463, 818-821. | 1.3 | 9 |
| 62 | Modelling the erosion/deposition pattern of the Tore Supra Toroidal Pumped Limiter. Journal of Nuclear Materials, 2015, 463, 827-831. | 1.3 | 1 |
| 63 | Material deposition on inner divertor quartz-micro balances during ITER-like wall operation in JET. Journal of Nuclear Materials, 2015, 463, 796-799. | 1.3 | 8 |
| 64 | Preliminary Monte Carlo simulation of beryllium migration during JET ITER-like wall divertor operation. Journal of Nuclear Materials, 2015, 463, 800-804. | 1.3 | 3 |
| 65 | Beryllium migration in JET ITER-like wall plasmas. Nuclear Fusion, 2015, 55, 063021. | 1.6 | 83 |
| 66 | Material migration studies with an ITER first wall panel proxy on EAST. Nuclear Fusion, 2015, 55, 023013. | 1.6 | 35 |
| 67 | Modeling of tungsten transport in the linear plasma device PSI-2 with the 3D Monte-Carlo code ERO. Journal of Nuclear Materials, 2015, 463, 268-271. | 1.3 | 7 |
| 68 | Kinetic modelling of material erosion and impurity transport in edge localized modes in EAST. Nuclear Fusion, 2015, 55, 043003. | 1.6 | 16 |
| 69 | Ion target impact energy during Type I edge localized modes in JET ITER-like Wall. Plasma Physics and Controlled Fusion, 2015, 57, 085006. | 0.9 | 44 |
| 70 | Modelling of surface evolution of rough surface on divertor target in fusion devices. Journal of Nuclear Materials, 2015, 463, 372-376. | 1.3 | 16 |
| 71 | Erosion/re-deposition modeling in an ITER divertor-like high-density, low-temperature plasma beam. Plasma Physics and Controlled Fusion, 2014, 56, 095028. | 0.9 | 1 |
| 72 | Modelling of surface roughness effects on impurity erosion and deposition in TEXTOR with a code package SURO/ERO/SDPIC. Nuclear Fusion, 2014, 54, 123015. | 1.6 | 17 |

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| 73 | Estimation of the contribution of gaps to tritium retention in the divertor of ITER. Physica Scripta, 2014, T159, 014063. | 1.2 | 4 |
| 74 | First results from the ^{10}Be marker experiment in JET with ITER-like wall. Nuclear Fusion, 2014, 54, 082004. | 1.6 | 4 |
| 75 | Study of physical and chemical assisted physical sputtering of beryllium in the JET ITER-like wall. Nuclear Fusion, 2014, 54, 103001. | 1.6 | 55 |
| 76 | Removable samples for ITER—a feasibility and conceptual study. Physica Scripta, 2014, T159, 014004. | 1.2 | 6 |
| 77 | Determination of Be sputtering yields from spectroscopic observations at the JET ITER-like wall based on three-dimensional ERO modelling. Physica Scripta, 2014, T159, 014057. | 1.2 | 21 |
| 78 | Simulation of spectroscopic patterns obtained in W/C test-limiter sputtering experiment at TEXTOR. Journal of Nuclear Materials, 2013, 438, S351-S355. | 1.3 | 6 |
| 79 | Kinetic effects of inclined magnetic field on physical sputtering by impurity ions. Journal of Nuclear Materials, 2013, 438, S909-S912. | 1.3 | 10 |
| 80 | Improved carbon migration modelling with the ERO code. Journal of Nuclear Materials, 2013, 438, S891-S894. | 1.3 | 2 |
| 81 | Material deposition and migration processes with resonant magnetic perturbation fields at TEXTOR. Journal of Nuclear Materials, 2013, 438, S602-S606. | 1.3 | 5 |
| 82 | Carbon transport and escape fraction in a high density plasma beam. Journal of Nuclear Materials, 2013, 438, S629-S632. | 1.3 | 5 |
| 83 | Carbon deposition at the bottom of gaps in TEXTOR experiments. Journal of Nuclear Materials, 2013, 438, S775-S779. | 1.3 | 5 |
| 84 | Modelling of lithium erosion and transport in FTU lithium experiments. Journal of Nuclear Materials, 2013, 438, S690-S693. | 1.3 | 9 |
| 85 | Spectroscopic measurements of Be erosion at JET ILW and interpretation with ERO modelling. Journal of Nuclear Materials, 2013, 438, S267-S271. | 1.3 | 26 |
| 86 | Modeling of divertor particle and heat loads during application of resonant magnetic perturbation fields for ELM control in ITER. Journal of Nuclear Materials, 2013, 438, S194-S198. | 1.3 | 25 |
| 87 | Studies of impurity migration in TEXTOR by local tracer injection. Journal of Nuclear Materials, 2013, 438, S723-S726. | 1.3 | 9 |
| 88 | Multiscale modeling of BeD release and transport in PISCES-B. Journal of Nuclear Materials, 2013, 438, S276-S279. | 1.3 | 15 |
| 89 | Molecules can be sputtered also from pure metals: sputtering of beryllium hydride by fusion plasma-wall interactions. Plasma Physics and Controlled Fusion, 2013, 55, 074004. | 0.9 | 29 |
| 90 | Modelling of local carbon deposition on a rough test limiter exposed to the edge plasma of TEXTOR. Plasma Physics and Controlled Fusion, 2013, 55, 055004. | 0.9 | 12 |

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| 91 | Global migration of impurities in tokamaks. Plasma Physics and Controlled Fusion, 2013, 55, 124029. | 0.9 | 15 |
| 92 | Modeling of Impurity Transport in the Divertor of JET. Plasma and Fusion Research, 2013, 8, 2402038-2402038. | 0.3 | 1 |
| 93 | Dissociative recombination and electron-impact de-excitation in CH photon emission under ITER divertor-relevant plasma conditions. Plasma Physics and Controlled Fusion, 2012, 54, 095013. | 0.9 | 15 |
| 94 | Outer divertor of ASDEX Upgrade in low-density L-mode discharges in forward and reversed magnetic field: II. Analysis of local impurity migration. Nuclear Fusion, 2012, 52, 103007. | 1.6 | 7 |
| 95 | Erosion and Deposition Mechanisms in Fusion Plasmas. Fusion Science and Technology, 2012, 61, 230-245. | 0.6 | 2 |
| 96 | Simulation of hydrogen retention and re-emission from tungsten exposed to divertor plasmas. Physica Scripta, 2011, T145, 014047. | 1.2 | 2 |
| 97 | Modelling of carbon deposition from CD ₄ injection in the far scrape-off layer of TEXTOR. Physica Scripta, 2011, T145, 014005. | 1.2 | 3 |
| 98 | ERO code benchmarking of ITER first wall beryllium erosion/re-deposition against LIM predictions. Physica Scripta, 2011, T145, 014008. | 1.2 | 38 |
| 99 | Passive protection of the ITER diagnostic mirrors. Physica Scripta, 2011, T145, 014071. | 1.2 | 10 |
| 100 | Studies of the influence of external hydrocarbon injection on local plasma conditions and resulting carbon transport. Journal of Nuclear Materials, 2011, 415, S270-S273. | 1.3 | 6 |
| 101 | PIC simulation of kinetic effects of plasma and consequences for physical sputtering. Journal of Nuclear Materials, 2011, 415, S192-S195. | 1.3 | 4 |
| 102 | Deposition and re-erosion studies by means of local impurity injection in TEXTOR. Journal of Nuclear Materials, 2011, 415, S239-S245. | 1.3 | 25 |
| 103 | Effect of E \times B driven transport on the deposition of carbon in the outer divertor of ASDEX Upgrade. Journal of Nuclear Materials, 2011, 415, S231-S234. | 1.3 | 12 |
| 104 | Analysis of the local re-deposition behavior of carbon at the main walls in TEXTOR by CD ₄ gas injection and Quartz Microbalance techniques. Journal of Nuclear Materials, 2011, 415, S246-S249. | 1.3 | 1 |
| 105 | Induced carbon deposition by local hydrocarbon injection into detached divertor plasmas in JET. Journal of Nuclear Materials, 2011, 415, S235-S238. | 1.3 | 1 |
| 106 | Simulation of Be \leftrightarrow C interaction dynamics in mixed Be/C layers formed in experiments at PISCES-B. Journal of Nuclear Materials, 2011, 415, S219-S222. | 1.3 | 10 |
| 107 | Overview of material migration and mixing, fuel retention and cleaning of ITER-like castellated structures in TEXTOR. Journal of Nuclear Materials, 2011, 415, S289-S292. | 1.3 | 20 |
| 108 | Quantification of chemical erosion in the DIII-D divertor and implications for ITER. Journal of Nuclear Materials, 2011, 415, S141-S144. | 1.3 | 2 |

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| 109 | Multi machine scaling of fuel retention in 4 carbon dominated tokamaks. Journal of Nuclear Materials, 2011, 415, S735-S739. | 1.3 | 20 |
| 110 | Particle-in-cell simulations of plasma interaction with shaped and unshaped gaps in TEXTOR. Plasma Physics and Controlled Fusion, 2011, 53, 115004. | 0.9 | 18 |
| 111 | Erosion and Deposition Mechanisms in Fusion Plasmas. Fusion Science and Technology, 2010, 57, 277-292. | 0.6 | 5 |
| 112 | Modelling of Impurity Transport in the Linear Plasma Devices PISCES-B and Pilot-PSI Using the Monte-Carlo Code ERO. Contributions To Plasma Physics, 2010, 50, 432-438. | 0.5 | 17 |
| 113 | Modelling of Carbon Transport in the Outer Divertor Plasma of ASDEX Upgrade. Contributions To Plasma Physics, 2010, 50, 439-444. | 0.5 | 6 |
| 114 | Analysis of Carbon Deposition on the FirstWall of LHD by Monte Carlo Simulation. Contributions To Plasma Physics, 2010, 50, 451-457. | 0.5 | 5 |
| 115 | Molecular dynamics and dynamic Monte Carlo studies of mixed materials and their impact on plasma wall interactions. Fusion Engineering and Design, 2010, 85, 1167-1172. | 1.0 | 10 |
| 116 | Modelling of local carbon deposition from methane and ethene injection through graphite and tungsten test limiters in TEXTOR. Plasma Physics and Controlled Fusion, 2010, 52, 045005. | 0.9 | 7 |
| 117 | Modelling of impurity deposition in gaps of castellated surfaces with the 3D-GAPS code. Plasma Physics and Controlled Fusion, 2010, 52, 075007. | 0.9 | 29 |
| 118 | Determination of rate coefficients for fusion-relevant atoms and molecules by modelling and measurement in the boundary layer of TEXTOR. Journal of Physics B: Atomic, Molecular and Optical Physics, 2010, 43, 144017. | 0.6 | 47 |
| 119 | Nonlinear Impact of Edge Localized Modes on Carbon Erosion in the Divertor of the JET Tokamak. Physical Review Letters, 2009, 102, 045007. | 2.9 | 27 |
| 120 | Simulation of light emission from hydrocarbon injection in TEXTOR using the ERO code. Plasma Physics and Controlled Fusion, 2009, 51, 055019. | 0.9 | 12 |
| 121 | Castellated structures for ITER: Differences of impurity deposition and fuel accumulation in the toroidal and poloidal gaps. Journal of Nuclear Materials, 2009, 386-388, 809-812. | 1.3 | 7 |
| 122 | Effects of tungsten surface conditions on carbon deposition. Journal of Nuclear Materials, 2009, 390-391, 44-48. | 1.3 | 19 |
| 123 | Investigations of castellated structures for ITER: The effect of castellation shaping and alignment on fuel retention and impurity deposition in gaps. Journal of Nuclear Materials, 2009, 390-391, 556-559. | 1.3 | 32 |
| 124 | Modelling of Be transport in PSI experiments at PISCES-B. Journal of Nuclear Materials, 2009, 390-391, 106-109. | 1.3 | 8 |
| 125 | Simulation of redeposition of carbon/hydrocarbon on a material surface with castellated structures. Journal of Nuclear Materials, 2009, 390-391, 119-122. | 1.3 | 10 |
| 126 | Estimations of erosion fluxes, material deposition and tritium retention in the divertor of ITER. Journal of Nuclear Materials, 2009, 390-391, 152-155. | 1.3 | 36 |

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| 127 | Improvement of surface processes modelling in the ERO code. Journal of Nuclear Materials, 2009, 390-391, 175-178. | 1.3 | 2 |
| 128 | Numerical modelling of steady-state fluxes at the ITER first wall. Journal of Nuclear Materials, 2009, 390-391, 528-531. | 1.3 | 23 |
| 129 | Recent analysis of key plasma wall interactions issues for ITER. Journal of Nuclear Materials, 2009, 390-391, 1-9. | 1.3 | 671 |
| 130 | Simulation of hydrocarbon reflection from carbon and tungsten surfaces and its impact on codeposition patterns on plasma facing components. Journal of Nuclear Materials, 2009, 390-391, 72-75. | 1.3 | 24 |
| 131 | Modeling of erosion and deposition by the Monte Carlo codes EDDY and ERO. Physica Scripta, 2009, T138, 014010. | 1.2 | 7 |
| 132 | Prediction of long-term tritium retention in the divertor of ITER: influence of modelling assumptions on retention rates. Physica Scripta, 2009, T138, 014011. | 1.2 | 18 |
| 133 | Chemical erosion of different carbon composites under ITER-relevant plasma conditions. Physica Scripta, 2009, T138, 014017. | 1.2 | 16 |
| 134 | ERO modelling of local deposition of injected ^{13}C tracer at the outer divertor of JET. Physica Scripta, 2009, T138, 014021. | 1.2 | 6 |
| 135 | Comparison of $^{13}\text{C}_2\text{H}_4$ and $^{13}\text{CH}_4$ injection through graphite and tungsten limiters in TEXTOR. Physica Scripta, 2009, T138, 014022. | 1.2 | 6 |
| 136 | Progress in Edge Plasma Transport Modeling on JET. Contributions To Plasma Physics, 2008, 48, 190-195. | 0.5 | 3 |
| 137 | Effect of surface roughness and substrate material on carbon erosion and deposition in the TEXTOR tokamak. Plasma Physics and Controlled Fusion, 2008, 50, 095008. | 0.9 | 47 |
| 138 | Modelling of $^{13}\text{CH}_4$ injection experiments with graphite and tungsten test limiters in TEXTOR using the coupled code ERO-SDTrimSP. Plasma Physics and Controlled Fusion, 2008, 50, 015006. | 0.9 | 31 |
| 139 | Modelling of carbon migration during JET ^{13}C injection experiments. Nuclear Fusion, 2008, 48, 105002. | 1.6 | 29 |
| 140 | Erosion and Deposition Mechanisms in Fusion Plasmas. Fusion Science and Technology, 2008, 53, 259-277. | 0.6 | 5 |
| 141 | Modelling of chemical erosion mitigation experiments at PISCES-B using the 3D Monte-Carlo code ERO. Physica Scripta, 2007, T128, 127-132. | 1.2 | 15 |
| 142 | Chapter 4: Power and particle control. Nuclear Fusion, 2007, 47, S203-S263. | 1.6 | 891 |
| 143 | Modelling of tritium retention and target lifetime of the ITER divertor using the ERO code. Journal of Nuclear Materials, 2007, 363-365, 91-95. | 1.3 | 56 |
| 144 | Study of local carbon transport on graphite, tungsten and molybdenum test limiters in TEXTOR by $^{13}\text{CH}_4$ tracer injection. Journal of Nuclear Materials, 2007, 363-365, 179-183. | 1.3 | 25 |

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| 145 | The effect of the magnetic topology on particle recycling in the ergodic divertor of TEXTOR. Journal of Nuclear Materials, 2007, 363-365, 377-381. | 1.3 | 2 |
| 146 | Diagnostic mirrors for ITER: A material choice and the impact of erosion and deposition on their performance. Journal of Nuclear Materials, 2007, 363-365, 1395-1402. | 1.3 | 94 |
| 147 | Carbon transport, deposition and fuel accumulation in castellated structures exposed in TEXTOR. Journal of Nuclear Materials, 2007, 367-370, 1481-1486. | 1.3 | 25 |
| 148 | Long-term erosion and deposition studies of the main graphite limiter in TEXTOR. Physica Scripta, 2007, T128, 35-39. | 1.2 | 5 |
| 149 | Castellated structures for ITER: the influence of the shape of castellation on the impurity deposition and fuel accumulation in gaps. Physica Scripta, 2007, T128, 45-49. | 1.2 | 10 |
| 150 | Tritium retention in next step devices and the requirements for mitigation and removal techniques. Plasma Physics and Controlled Fusion, 2006, 48, B189-B199. | 0.9 | 83 |
| 151 | Impurity Transport Modelling in Edge Plasmas of Fusion Devices with the Monte Carlo Code ERO. Contributions To Plasma Physics, 2006, 46, 628-634. | 0.5 | 13 |
| 152 | Investigation of carbon transport by $^{13}\text{CH}_4$ injection through graphite and tungsten test limiters in TEXTOR. Plasma Physics and Controlled Fusion, 2006, 48, 1401-1412. | 0.9 | 29 |
| 153 | Overview of Erosion Mechanisms, Impurity Transport, and Deposition in TEXTOR and Related Modeling. Fusion Science and Technology, 2005, 47, 146-160. | 0.6 | 7 |
| 154 | Mixed and High-Z Plasma-Facing Materials in TEXTOR. Springer Series in Chemical Physics, 2005, , 319-333. | 0.2 | 2 |
| 155 | Carbon chemical erosion in H-mode discharges in ASDEX Upgrade divertor IIb: flux dependence and local redeposition. Journal of Nuclear Materials, 2005, 337-339, 985-989. | 1.3 | 17 |
| 156 | Experimental observations and modelling of carbon transport in the inner divertor of JET. Journal of Nuclear Materials, 2005, 337-339, 17-24. | 1.3 | 17 |
| 157 | Identification of molecular carbon sources in the JET divertor by means of emission spectroscopy. Journal of Nuclear Materials, 2005, 337-339, 1058-1063. | 1.3 | 33 |
| 158 | Flux dependence of carbon erosion and implication for ITER. Journal of Nuclear Materials, 2005, 337-339, 970-974. | 1.3 | 90 |
| 159 | Dynamic transition between erosion and deposition on a tungsten surface exposed to edge plasmas containing carbon impurities. Journal of Nuclear Materials, 2005, 337-339, 882-886. | 1.3 | 9 |
| 160 | Report on the 11th European Fusion Physics Workshop (Heraklion, Crete, 8-10 December 2003). Plasma Physics and Controlled Fusion, 2005, 47, 1351-1366. | 0.9 | 0 |
| 161 | Toroidal Plasma Rotation Induced by the Dynamic Ergodic Divertor in the TEXTOR Tokamak. Physical Review Letters, 2005, 94, 015003. | 2.9 | 73 |
| 162 | Flux dependence of carbon chemical erosion by deuterium ions. Nuclear Fusion, 2004, 44, L21-L25. | 1.6 | 97 |

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| 163 | Modelling of carbon transport in fusion devices: evidence of enhanced re-erosion of in-situ re-deposited carbon. <i>Journal of Nuclear Materials</i> , 2004, 328, 62-66. | 1.3 | 56 |
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