Andreas Kirschner

List of Publications by Year
in descending order

190	5,430	136740	102304
papers	citations	32 h-index	66 g-index

191	191	191	2438
all docs	docs citations	times ranked	citing authors

#	Article	IF	Citations
1	Chapter 4: Power and particle control. Nuclear Fusion, 2007, 47, S203-S263.	1.6	891
2	Recent analysis of key plasma wall interactions issues for ITER. Journal of Nuclear Materials, 2009, 390-391, 1-9.	1.3	671
3	Simulation of the plasma-wall interaction in a tokamak with the Monte Carlo code ERO-TEXTOR. Nuclear Fusion, 2000, 40, 989-1001.	1.6	233
4	Overview of first Wendelstein 7-X high-performance operation. Nuclear Fusion, 2019, 59, 112004.	1.6	165
5	Flux dependence of carbon chemical erosion by deuterium ions. Nuclear Fusion, 2004, 44, L21-L25.	1.6	97
6	Diagnostic mirrors for ITER: A material choice and the impact of erosion and deposition on their performance. Journal of Nuclear Materials, 2007, 363-365, 1395-1402.	1.3	94
7	Flux dependence of carbon erosion and implication for ITER. Journal of Nuclear Materials, 2005, 337-339, 970-974.	1.3	90
8	Tritium retention in next step devices and the requirements for mitigation and removal techniques. Plasma Physics and Controlled Fusion, 2006, 48, B189-B199.	0.9	83
9	Beryllium migration in JET ITER-like wall plasmas. Nuclear Fusion, 2015, 55, 063021.	1.6	83
10	Plasmaâ€“wall interaction studies within the EUROfusion consortium: progress on plasma-facing components development and qualification. Nuclear Fusion, 2017, 57, 116041.	1.6	75
11	Toroidal Plasma Rotation Induced by the Dynamic Ergodic Divertor in the TEXTOR Tokamak. Physical Review Letters, 2005, 94, 015003.	2.9	73
12	Erosion, screening, and migration of tungsten in the JET divertor. Nuclear Fusion, 2019, 59, 096035.	1.6	60
13	Erosion and redeposition of wall material in controlled fusion devices. Vacuum, 2002, 67, 399-408.	1.6	56
14	Modelling of carbon transport in fusion devices: evidence of enhanced re-erosion of in-situ re-deposited carbon. Journal of Nuclear Materials, 2004, 328, 62-66.	1.3	56
15	Modelling of tritium retention and target lifetime of the ITER divertor using the ERO code. Journal of Nuclear Materials, 2007, 363-365, 91-95.	1.3	56
16	Investigation of carbon transport in the scrape-off layer of TEXTOR-94. Journal of Nuclear Materials, 2001, 290-293, 362-366.	1.3	55
17	Study of physical and chemical assisted physical sputtering of beryllium in the JET ITER-like wall. Nuclear Fusion, 2014, 54, 103001.	1.6	55
18	Short and long range transport of materials eroded from wall components in fusion devices. Journal of Nuclear Materials, 2003, 313-316, 311-320.	1.3	49

#	Article	IF	Citations
19	Effect of surface roughness and substrate material on carbon erosion and deposition in the TEXTOR tokamak. Plasma Physics and Controlled Fusion, 2008, 50, 095008.	0.9	47
20	Determination of rate coefficients for fusion-relevant atoms and molecules by modelling and measurement in the boundary layer of TEXTOR. Journal of Physics B: Atomic, Molecular and Optical Physics, 2010, 43, 144017.	0.6	47
21	Experiments with tungsten limiters in TEXTOR-94. Journal of Nuclear Materials, 1998, 258-263, 858-864.	1.3	46
22	Ion target impact energy during Type I edge localized modes in JET ITER-like Wall. Plasma Physics and Controlled Fusion, 2015, 57, 085006.	0.9	44
23	Operation of TEXTOR-94 with tungsten poloidal main limiters. Journal of Nuclear Materials, 2001, 290-293, 947-952.	1.3	42
24	Simulation of gross and net erosion of high-Z materials in the DIII-D divertor. Nuclear Fusion, 2016, 56, 016021.	1.6	41
25	Operating a full tungsten actively cooled tokamak: overview of WEST first phase of operation. Nuclear Fusion, 2022, 62, 042007.	1.6	39
26	ERO code benchmarking of ITER first wall beryllium erosion/re-deposition against LIM predictions. Physica Scripta, 2011, T145, 014008.	1.2	38
27	Estimations of erosion fluxes, material deposition and tritium retention in the divertor of ITER. Journal of Nuclear Materials, 2009, 390-391, 152-155.	1.3	36
28	Beryllium global erosion and deposition at JET-ILW simulated with ERO2.0. Nuclear Materials and Energy, 2019, 18, 331-338.	0.6	36
29	Material migration studies with an ITER first wall panel proxy on EAST. Nuclear Fusion, 2015, 55, 023013.	1.6	35
30	The dynamic ergodic divertor in the TEXTOR tokamak: plasma response to dynamic helical magnetic field perturbations. Plasma Physics and Controlled Fusion, 2004, 46, B143-B155.	0.9	34
31	Three-dimensional modeling of plasma edge transport and divertor fluxes during application of resonant magnetic perturbations on ITER. Nuclear Fusion, 2016, 56, 066008.	1.6	34
32	First divertor physics studies in Wendelstein 7-X. Nuclear Fusion, 2019, 59, 096014.	1.6	34
33	Identification of molecular carbon sources in the JET divertor by means of emission spectroscopy. Journal of Nuclear Materials, 2005, 337-339, 1058-1063.	1.3	33
34	Investigations of castellated structures for ITER: The effect of castellation shaping and alignment on fuel retention and impurity deposition in gaps. Journal of Nuclear Materials, 2009, 390-391, 556-559.	1.3	32
35	Modelling of13CH4injection experiments with graphite and tungsten test limiters in TEXTOR using the coupled code ERO-SDTrimSP. Plasma Physics and Controlled Fusion, 2008, 50, 015006.	0.9	31
36	Experimental estimation of tungsten impurity sputtering due to Type I ELMs in JET-ITER-like wall using pedestal electron cyclotron emission and target Langmuir probe measurements. Physica Scripta, 2016, T167, 014005.	1.2	31

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37	Modelling of plasma-wall interaction and impurity transport in fusion devices and prompt deposition of tungsten as application. Plasma Physics and Controlled Fusion, 2018, 60, 014041.	0.9	31
38	Investigation of carbon transport by13CH4injection through graphite and tungsten test limiters in TEXTOR. Plasma Physics and Controlled Fusion, 2006, 48, 1401-1412.	0.9	29
39	Modelling of carbon migration during JET13C injection experiments. Nuclear Fusion, 2008, 48, 105002.	1.6	29
40	Modelling of impurity deposition in gaps of castellated surfaces with the 3D-GAPS code. Plasma Physics and Controlled Fusion, 2010, 52, 075007.	0.9	29
41	Molecules can be sputtered also from pure metals: sputtering of beryllium hydride by fusion plasmaâ€“wall interactions. Plasma Physics and Controlled Fusion, 2013, 55, 074004.	0.9	29
42	Hydrocarbon transport in the MkIIa divertor of JET. Plasma Physics and Controlled Fusion, 2003, 45, 309-319.	0.9	27
43	Nonlinear Impact of Edge Localized Modes on Carbon Erosion in the Divertor of the JET Tokamak. Physical Review Letters, 2009, 102, 045007.	2.9	27
44	First ERO2.0 modeling of Be erosion and non-local transport in JET ITER-like wall. Physica Scripta, 2017, T170, 014018.	1.2	27
45	Surface roughness effect on Mo physical sputtering and re-deposition in the linear plasma device PSI-2 predicted by ERO2.0. Nuclear Materials and Energy, 2019, 19, 13-18.	0.6	27
46	Spectroscopic measurements of Be erosion at JET ILW and interpretation with ERO modelling. Journal of Nuclear Materials, 2013, 438, S267-S271.	1.3	26
47	Modelling of the material transport and layer formation in the divertor of JET: Comparison of ITER-like wall with full carbon wall conditions. Journal of Nuclear Materials, 2015, 463, 116-122.	1.3	26
48	An Analytical Expression for the Electric Field and Particle Tracing in Modelling of Be Erosion Experiments at the JET ITERâ€like Wall. Contributions To Plasma Physics, 2016, 56, 640-645.	0.5	26
49	Chemical Erosion Measurements in Tokamaks by Spectroscopy. Physica Scripta, 2004, T111, 42.	1.2	25
50	Study of local carbon transport on graphite, tungsten and molybdenum test limiters in TEXTOR by 13CH4 tracer injection. Journal of Nuclear Materials, 2007, 363-365, 179-183.	1.3	25
51	Carbon transport, deposition and fuel accumulation in castellated structures exposed in TEXTOR. Journal of Nuclear Materials, 2007, 367-370, 1481-1486.	1.3	25
52	Deposition and re-erosion studies by means of local impurity injection in TEXTOR. Journal of Nuclear Materials, 2011, 415, S239-S245.	1.3	25
53	Modeling of divertor particle and heat loads during application of resonant magnetic perturbation fields for ELM control in ITER. Journal of Nuclear Materials, 2013, 438, S194-S198.	1.3	25
54	Fuel inventory and deposition in castellated structures in JET-ILW. Nuclear Fusion, 2017, 57, 066027.	1.6	25

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55	Advances in the modeling of chemical erosion/redeposition of carbon divertors and application to the JET tritium codeposition problem. Journal of Nuclear Materials, 2003, 313-316, 424-428.	1.3	24
56	Simulation of hydrocarbon reflection from carbon and tungsten surfaces and its impact on codeposition patterns on plasma facing components. Journal of Nuclear Materials, 2009, 390-391, 72-75.	1.3	24
57	Modelling of tungsten erosion and deposition in the divertor of JET-ILW in comparison to experimental findings. Nuclear Materials and Energy, 2019, 18, 239-244.	0.6	24
58	Experimental confirmation of efficient island divertor operation and successful neoclassical transport optimization in Wendelstein 7-X. Nuclear Fusion, 2022, 62, 042022.	1.6	24
59	Numerical modelling of steady-state fluxes at the ITER first wall. Journal of Nuclear Materials, 2009, 390-391, 528-531.	1.3	23
60	Overview of wall probes for erosion and deposition studies in the TEXTOR tokamak. Matter and Radiation at Extremes, 2017, 2, 87-104.	1.5	23
61	Modelling of erosion and deposition at limiter surfaces and divertor target plates. Journal of Nuclear Materials, 2001, 290-293, 238-244.	1.3	22
62	Determination of Be sputtering yields from spectroscopic observations at the JET ITER-like wall based on three-dimensional ERO modelling. Physica Scripta, 2014, T159, 014057.	1.2	21
63	Modelling of Impurity Transport and Plasmaâ€“Wall Interaction in Fusion Devices with the ERO Code: Basics of the Code and Examples of Application. Contributions To Plasma Physics, 2016, 56, 622-627.	0.5	21
64	ERO modelling of tungsten erosion in the linear plasma device PSI-2. Nuclear Materials and Energy, 2017, 12, 253-260.	0.6	21
65	Chemical erosion behaviour of carbon materials in fusion devices. Journal of Nuclear Materials, 2003, 313-316, 354-359.	1.3	20
66	Overview of material migration and mixing, fuel retention and cleaning of ITER-like castellated structures in TEXTOR. Journal of Nuclear Materials, 2011, 415, S289-S292.	1.3	20
67	Multi machine scaling of fuel retention in 4 carbon dominated tokamaks. Journal of Nuclear Materials, 2011, 415, S735-S739.	1.3	20
68	ERO modelling of tungsten erosion and re-deposition in EAST L mode discharges. Physics of Plasmas, 2017, 24, 092512.	0.7	20
69	Effects of tungsten surface conditions on carbon deposition. Journal of Nuclear Materials, 2009, 390-391, 44-48.	1.3	19
70	ERO2.0 modelling of the effects of surface roughness on molybdenum erosion and redeposition in the PSI-2 linear plasma device. Physica Scripta, 2020, T171, 014057.	1.2	19
71	Prediction of long-term tritium retention in the divertor of ITER: influence of modelling assumptions on retention rates. Physica Scripta, 2009, T138, 014011.	1.2	18
72	Particle-in-cell simulations of plasma interaction with shaped and unshaped gaps in TEXTOR. Plasma Physics and Controlled Fusion, 2011, 53, 115004.	0.9	18

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73	Carbon chemical erosion in H-mode discharges in ASDEX Upgrade divertor IIb: flux dependence and local redeposition. Journal of Nuclear Materials, 2005, 337-339, 985-989.	1.3	17
74	Experimental observations and modelling of carbon transport in the inner divertor of JET. Journal of Nuclear Materials, 2005, 337-339, 17-24.	1.3	17
75	Modelling of Impurity Transport in the Linear Plasma Devices PISCESâ€B and Pilotâ€PSI Using the Monteâ€€arlo Code ERO. Contributions To Plasma Physics, 2010, 50, 432-438.	0.5	17
76	Modelling of surface roughness effects on impurity erosion and deposition in TEXTOR with a code package SURO/ERO/SDPIC. Nuclear Fusion, 2014, 54, 123015.	1.6	17
77	Improved ERO modelling for spectroscopy of physically and chemically assisted eroded beryllium from the JET-ILW. Nuclear Materials and Energy, 2016, 9, 604-609.	0.6	17
78	First Monteâ€€arlo modelling of global beryllium migration in ITER using ERO2.0. Contributions To Plasma Physics, 2020, 60, e201900149.	0.5	17
79	Investigation of Erosion and Deposition on Wall Components of TEXTOR-94. Physica Scripta, 1999, T81, 19.	1.2	16
80	Chemical erosion of different carbon composites under ITER-relevant plasma conditions. Physica Scripta, 2009, T138, 014017.	1.2	16
81	Kinetic modelling of material erosion and impurity transport in edge localized modes in EAST. Nuclear Fusion, 2015, 55, 043003.	1.6	16
82	Modelling of surface evolution of rough surface on divertor target in fusion devices. Journal of Nuclear Materials, 2015, 463, 372-376.	1.3	16
83	Advances in understanding of high-Zmaterial erosion and re-deposition in low-Zwall environment in DIII-D. Nuclear Fusion, 2017, 57, 056016.	1.6	16
84	Determination of tungsten sources in the JET-ILW divertor by spectroscopic imaging in the presence of a strong plasma continuum. Nuclear Materials and Energy, 2019, 18, 118-124.	0.6	16
85	First efforts in numerical modeling of tungsten migration in WEST with SolEdge2D-EIRENE and ERO2.0. Physica Scripta, 2020, T171, 014013.	1.2	16
86	Comparison of impurity production, recycling and power deposition on carbon and tungsten limiters in TEXTOR-94. Journal of Nuclear Materials, 2001, 290-293, 276-280.	1.3	15
87	Tomographic reconstruction of 2D line radiation distribution in the JET MkIIGB divertor. Journal of Nuclear Materials, 2003, 313-316, 925-930.	1.3	15
88	Modelling of chemical erosion mitigation experiments at PISCES-B using the 3D Monte-Carlo code ERO. Physica Scripta, 2007, T128, 127-132.	1.2	15
89	Dissociative recombination and electron-impact de-excitation in CH photon emission under ITER divertor-relevant plasma conditions. Plasma Physics and Controlled Fusion, 2012, 54, 095013.	0.9	15
90	Multiscale modeling of BeD release and transport in PISCES-B. Journal of Nuclear Materials, 2013, 438, S276-S279.	1.3	15

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91	Global migration of impurities in tokamaks. Plasma Physics and Controlled Fusion, 2013, 55, 124029.	0.9	15
92	Improved ERO modelling of beryllium erosion at ITER upper first wall panel using JET-ILW and PISCES-B experience. Nuclear Materials and Energy, 2019, 19, 510-515.	0.6	15
93	Graphiteâ€“tungsten twin limiters in studies of material mixing processes on high heat flux components. Journal of Nuclear Materials, 2000, 283-287, 1089-1093.	1.3	14
94	Modelling of the transport of methane and higher hydrocarbons in fusion devices. Journal of Nuclear Materials, 2003, 313-316, 444-449.	1.3	14
95	Deposition in the inner and outer corners of the JET divertor with carbon wall and metallic ITER-like wall. Physica Scripta, 2016, T167, 014052.	1.2	14
96	Estimates of RF-induced erosion at antenna-connected beryllium plasma-facing components in JET. Physica Scripta, 2016, T167, 014035.	1.2	14
97	Chemical Erosion in TEXTOR-94. Physica Scripta, 1999, T81, 48.	1.2	14
98	Impurity Transport Modelling in Edge Plasmas of Fusion Devices with the Monte Carlo Code ERO. Contributions To Plasma Physics, 2006, 46, 628-634.	0.5	13
99	Beryllium erosion and redeposition in ITER H, He and Dâ€“T discharges. Nuclear Fusion, 2022, 62, 036011.	1.6	13
100	Modeling of Material Mixing Effects on Plasma Surface Interactions in Magnetic Fusion Devices. Physica Scripta, 2004, T111, 138.	1.2	12
101	Simulation of light emission from hydrocarbon injection in TEXTOR using the ERO code. Plasma Physics and Controlled Fusion, 2009, 51, 055019.	0.9	12
102	Effect of EÃ—B driven transport on the deposition of carbon in the outer divertor of ASDEX Upgrade. Journal of Nuclear Materials, 2011, 415, S231-S234.	1.3	12
103	Modelling of local carbon deposition on a rough test limiter exposed to the edge plasma of TEXTOR. Plasma Physics and Controlled Fusion, 2013, 55, 055004.	0.9	12
104	Latest results of Eurofusion plasma-facing components research in the areas of power loading, material erosion and fuel retention. Nuclear Fusion, 2022, 62, 042013.	1.6	11
105	An overview of JET edge modelling activities. Journal of Nuclear Materials, 2003, 313-316, 868-872.	1.3	10
106	Simulation of redeposition of carbon/hydrocarbon on a material surface with castellated structures. Journal of Nuclear Materials, 2009, 390-391, 119-122.	1.3	10
107	Molecular dynamics and dynamic Monte Carlo studies of mixed materials and their impact on plasma wall interactions. Fusion Engineering and Design, 2010, 85, 1167-1172.	1.0	10
108	Passive protection of the ITER diagnostic mirrors. Physica Scripta, 2011, T145, 014071.	1.2	10

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109	Simulation of Beâ€“C interaction dynamics in mixed Be/C layers formed in experiments at PISCES-B. Journal of Nuclear Materials, 2011, 415, S219-S222.	1.3	10
110	Kinetic effects of inclined magnetic field on physical sputtering by impurity ions. Journal of Nuclear Materials, 2013, 438, S909-S912.	1.3	10
111	An analytical expression for ion velocities at the wall including the sheath electric field and surface biasing for erosion modeling at JET ILW. Nuclear Materials and Energy, 2017, 12, 341-345.	0.6	10
112	Castellated structures for ITER: the influence of the shape of castellation on the impurity deposition and fuel accumulation in gaps. Physica Scripta, 2007, T128, 45-49.	1.2	10
113	Plasma-wall interaction studies in W7-X: main results from the recent divertor operations. Physica Scripta, 2021, 96, 124059.	1.2	10
114	Deposition and Erosion in Local Shadow Regions of TEXTOR-94. Physica Scripta, 2001, T94, 141.	1.2	9
115	Deposition of Hydrogen Rich Carbon Films in Pump Ducts of TEXTOR. Physica Scripta, 2004, T111, 118.	1.2	9
116	Dynamic transition between erosion and deposition on a tungsten surface exposed to edge plasmas containing carbon impurities. Journal of Nuclear Materials, 2005, 337-339, 882-886.	1.3	9
117	Modelling of lithium erosion and transport in FTU lithium experiments. Journal of Nuclear Materials, 2013, 438, S690-S693.	1.3	9
118	Studies of impurity migration in TEXTOR by local tracer injection. Journal of Nuclear Materials, 2013, 438, S723-S726.	1.3	9
119	Analysis of rotating collectors from the private region of JET with carbon wall and metallic ITER-like wall. Journal of Nuclear Materials, 2015, 463, 818-821.	1.3	9
120	Surface roughness effects on plasma near a divertor plate and local impact angle. Nuclear Materials and Energy, 2017, 12, 313-317.	0.6	9
121	ERO modeling and sensitivity analysis of locally enhanced beryllium erosion by magnetically connected antennas. Nuclear Fusion, 2018, 58, 016046.	1.6	9
122	Review on global migration, fuel retention and modelling after TEXTOR decommission. Nuclear Materials and Energy, 2018, 17, 83-112.	0.6	9
123	A sensitivity analysis of numerical predictions for beryllium erosion and migration in ITER. Nuclear Materials and Energy, 2021, 26, 100904.	0.6	9
124	The impact of surface morphology on the erosion of metallic surfaces â€“ Modelling with the 3D Monte-Carlo code ERO2.0. Nuclear Materials and Energy, 2021, 27, 100987.	0.6	9
125	Local migration studies of high-Zmetals in the TEXTOR tokamak. Physica Scripta, 2016, T167, 014058.	1.2	9
126	Modeling of erosion and deposition patterns on Câ€“W and Wâ€“Ta twin limiters exposed to the TEXTOR edge plasmas. Journal of Nuclear Materials, 2004, 329-333, 732-736.	1.3	8

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127	Modelling of Be transport in PSI experiments at PISCES-B. Journal of Nuclear Materials, 2009, 390-391, 106-109.	1.3	8
128	Material deposition on inner divertor quartz-micro balances during ITER-like wall operation in JET. Journal of Nuclear Materials, 2015, 463, 796-799.	1.3	8
129	Overview of Erosion Mechanisms, Impurity Transport, and Deposition in TEXTOR and Related Modeling. Fusion Science and Technology, 2005, 47, 146-160.	0.6	7
130	Castellated structures for ITER: Differences of impurity deposition and fuel accumulation in the toroidal and poloidal gaps. Journal of Nuclear Materials, 2009, 386-388, 809-812.	1.3	7
131	Modeling of erosion and deposition by the Monte Carlo codes EDDY and ERO. Physica Scripta, 2009, T138, 014010.	1.2	7
132	Modelling of local carbon deposition from methane and ethene injection through graphite and tungsten test limiters in TEXTOR. Plasma Physics and Controlled Fusion, 2010, 52, 045005.	0.9	7
133	Outer divertor of ASDEX Upgrade in low-density L-mode discharges in forward and reversed magnetic field: II. Analysis of local impurity migration. Nuclear Fusion, 2012, 52, 103007.	1.6	7
134	Modeling of tungsten transport in the linear plasma device PSI-2 with the 3D Monte-Carlo code ERO. Journal of Nuclear Materials, 2015, 463, 268-271.	1.3	7
135	ERO modeling of beryllium erosion by helium plasma in experiments at PISCES-B. Nuclear Materials and Energy, 2017, 12, 1157-1162.	0.6	7
136	Numerical and analytic study of rough surface morphology on the angular distribution of eroded impurity. Contributions To Plasma Physics, 2017, 57, 329-335.	0.5	7
137	13C tracer deposition in EAST D and He plasmas investigated by high-throughput deuteron nuclear reaction analysis mapping. Nuclear Materials and Energy, 2020, 25, 100805.	0.6	7
138	Erosion and screening of tungsten during inter/intra-ELM periods in the JET-ILW divertor. Nuclear Materials and Energy, 2020, 25, 100859.	0.6	7
139	Modelling of Carbon Transport in the Outer Divertor Plasma of ASDEX Upgrade. Contributions To Plasma Physics, 2010, 50, 439-444.	0.5	6
140	Studies of the influence of external hydrocarbon injection on local plasma conditions and resulting carbon transport. Journal of Nuclear Materials, 2011, 415, S270-S273.	1.3	6
141	Simulation of spectroscopic patterns obtained in W/C test-limiter sputtering experiment at TEXTOR. Journal of Nuclear Materials, 2013, 438, S351-S355.	1.3	6
142	Removable samples for ITERâ€"a feasibility and conceptual study. Physica Scripta, 2014, T159, 014004.	1.2	6
143	Whole-machine material migration studies in the TEXTOR tokamak with molybdenum. Nuclear Materials and Energy, 2017, 12, 518-523.	0.6	6
144	Quartz micro-balance results of pulse-resolved erosion/deposition in the JET-ILW divertor. Nuclear Materials and Energy, 2017, 12, 478-482.	0.6	6

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145	ERO modelling of local deposition of injected13C tracer at the outer divertor of JET. Physica Scripta, 2009, T138, 014021.	1.2	6
146	Comparison of13C2H4and13CH4injection through graphite and tungsten limiters in TEXTOR. Physica Scripta, 2009, T138, 014022.	1.2	6
147	Understanding tungsten erosion during inter/intra-ELM periods in He-dominated JET-ILW plasmas. Physica Scripta, 0, , .	1.2	6
148	Monte-Carlo Simulations of Chemical Erosion in TEXTOR-94 with the ERO-TEXTOR Code. Physica Scripta, 2001, T91, 57.	1.2	5
149	Influence of Methane Fuelling on the 2-D Line Radiation Distribution in the JET MkIIGB Divertor. Physica Scripta, 2004, T111, 101.	1.2	5
150	Erosion and Deposition Mechanisms in Fusion Plasmas. Fusion Science and Technology, 2008, 53, 259-277.	0.6	5
151	Erosion and Deposition Mechanisms in Fusion Plasmas. Fusion Science and Technology, 2010, 57, 277-292.	0.6	5
152	Analysis of Carbon Deposition on the FirstWall of LHD by Monte Carlo Simulation. Contributions To Plasma Physics, 2010, 50, 451-457.	0.5	5
153	Material deposition and migration processes with resonant magnetic perturbation fields at TEXTOR. Journal of Nuclear Materials, 2013, 438, S602-S606.	1.3	5
154	Carbon transport and escape fraction in a high density plasma beam. Journal of Nuclear Materials, 2013, 438, S629-S632.	1.3	5
155	Carbon deposition at the bottom of gaps in TEXTOR experiments. Journal of Nuclear Materials, 2013, 438, S775-S779.	1.3	5
156	Long-term erosion and deposition studies of the main graphite limiter in TEXTOR. Physica Scripta, 2007, T128, 35-39.	1.2	5
157	Behaviour of Silicon-Doped CFC Limiter under High Heat Load in TEXTOR-94. Physica Scripta, 2001, T91, 61.	1.2	5
158	PIC simulation of kinetic effects of plasma and consequences for physical sputtering. Journal of Nuclear Materials, 2011, 415, S192-S195.	1.3	4
159	Estimation of the contribution of gaps to tritium retention in the divertor of ITER. Physica Scripta, 2014, T159, 014063.	1.2	4
160	First results from the¹⁰Be marker experiment in JET with ITER-like wall. Nuclear Fusion, 2014, 54, 082004.	1.6	4
161	Modelling of deposition and erosion of injected WF6 and MoF6 in TEXTOR. Nuclear Materials and Energy, 2017, 12, 564-568.	0.6	4
162	High-Z material erosion and its control in DIII-D carbon divertor. Nuclear Materials and Energy, 2017, 12, 247-252.	0.6	4

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163	ERO modeling of Cr sputtering in the linear plasma device PSI-2. Physica Scripta, 2017, T170, 014051.	1.2	4
164	Boron transport simulation using the ERO2.0 code for real-time wall conditioning in the large helical device. Nuclear Materials and Energy, 2020, 25, 100853.	0.6	4
165	Interpretative modeling of impurity transport and tungsten sources in WEST boundary plasma. Nuclear Fusion, 2021, 61, 126015.	1.6	4
166	Progress in Edge Plasma Transport Modeling on JET. Contributions To Plasma Physics, 2008, 48, 190-195.	0.5	3
167	Modelling of carbon deposition from CD₄injection in the far scrape-off layer of TEXTOR. Physica Scripta, 2011, T145, 014005.	1.2	3
168	Preliminary Monte Carlo simulation of beryllium migration during JET ITER-like wall divertor operation. Journal of Nuclear Materials, 2015, 463, 800-804.	1.3	3
169	Plasma-wall interactions in the presence of plasma fluctuationsâ€"interpretation of line emission from sputtered tungsten in PSI-2. Physica Scripta, 2017, T170, 014039.	1.2	3
170	Mixed and High-Z Plasma-Facing Materials in TEXTOR. Springer Series in Chemical Physics, 2005, , 319-333.	0.2	2
171	The effect of the magnetic topology on particle recycling in the ergodic divertor of TEXTOR. Journal of Nuclear Materials, 2007, 363-365, 377-381.	1.3	2
172	Improvement of surface processes modelling in the ERO code. Journal of Nuclear Materials, 2009, 390-391, 175-178.	1.3	2
173	Simulation of hydrogen retention and re-emission from tungsten exposed to divertor plasmas. Physica Scripta, 2011, T145, 014047.	1.2	2
174	Quantification of chemical erosion in the DIII-D divertor and implications for ITER. Journal of Nuclear Materials, 2011, 415, S141-S144.	1.3	2
175	Erosion and Deposition Mechanisms in Fusion Plasmas. Fusion Science and Technology, 2012, 61, 230-245.	0.6	2
176	Improved carbon migration modelling with the ERO code. Journal of Nuclear Materials, 2013, 438, S891-S894.	1.3	2
177	Simulation of Impurity Transport and Deposition in the Closed Helical Divertor in the Large Helical Device. Plasma and Fusion Research, 2021, 16, 2403004-2403004.	0.3	2
178	Fuel inventory and impurity deposition in castellated tungsten tiles in KSTAR: experiment and modelling. Physica Scripta, 2020, T171, 014049.	1.2	2
179	Predictive 3D modelling of erosion and deposition in ITER with ERO2.0: from beryllium main wall, tungsten divertor to full-tungsten device. Physica Scripta, 2022, 97, 014001.	1.2	2
180	Analysis of the local re-deposition behavior of carbon at the main walls in TEXTOR by CD4 gas injection and Quartz Microbalance techniques. Journal of Nuclear Materials, 2011, 415, S246-S249.	1.3	1

#	Article	IF	Citations
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