

Oscar Luis Cabellos De Francisco

List of Publications by Year in descending order

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Version: 2024-02-01

39
papers

598
citations

759233

12
h-index

642732

23
g-index

44
all docs

44
docs citations

44
times ranked

394
citing authors

#	ARTICLE	IF	CITATIONS
1	Determination of the uncertainties associated to the use of different nuclear data libraries in the analysis of extended-range rem-meters. Applied Radiation and Isotopes, 2022, 179, 110012.	1.5	5
2	Which nuclear data can be validated with LLNL pulsed-sphere experiments?. Annals of Nuclear Energy, 2021, 159, 108345.	1.8	13
3	Informing nuclear physics via machine learning methods with differential and integral experiments. Physical Review C, 2021, 104, .	2.9	14
4	WPEC Subgroup 44 computational Inter-comparison exercise on correlations in nuclear data libraries. Annals of Nuclear Energy, 2021, 164, 108605.	1.8	5
5	CIELO Collaboration Summary Results: International Evaluations of Neutron Reactions on Uranium, Plutonium, Iron, Oxygen and Hydrogen. Nuclear Data Sheets, 2018, 148, 189-213.	2.2	73
6	Checking, processing and verification of nuclear data covariances. EPJ Nuclear Sciences & Technologies, 2018, 4, 39.	0.7	2
7	A comparison of uncertainty propagation techniques using NDaST: full, half or zero Monte Carlo?. EPJ Nuclear Sciences & Technologies, 2018, 4, 14.	0.7	5
8	Nuclear Data Uncertainties for Typical LWR Fuel Assemblies and a Simple Reactor Core. Nuclear Data Sheets, 2017, 139, 1-76.	2.2	38
9	SFCOMPO-2.0: An OECD NEA database of spent nuclear fuel isotopic assays, reactor design specifications, and operating data. Annals of Nuclear Energy, 2017, 110, 779-788.	1.8	66
10	Nuclear data for fusion technology – the European approach. EPJ Web of Conferences, 2017, 146, 09003.	0.3	4
11	Nuclear data sensitivity and uncertainty analysis of effective neutron multiplication factor in various MYRRHA core configurations. Annals of Nuclear Energy, 2017, 101, 330-338.	1.8	21
12	Verification of the databases EXFOR and ENDF. EPJ Web of Conferences, 2017, 146, 06030.	0.3	2
13	EXFOR – a global experimental nuclear reaction data repository: Status and new developments. EPJ Web of Conferences, 2017, 146, 07003.	0.3	18
14	Improving nuclear data accuracy of ^{241}Am and ^{237}Np capture cross sections. EPJ Web of Conferences, 2017, 146, 11035.	0.3	3
15	Neutron-induced nuclear data for the MYRRHA fast spectrum facility. EPJ Web of Conferences, 2017, 146, 09007.	0.3	1
16	JANIS: NEA Java-based Nuclear Data Information System. EPJ Web of Conferences, 2017, 146, 07006.	0.3	13
17	Generation of fission yield covariances to correct discrepancies in the nuclear data libraries. Annals of Nuclear Energy, 2016, 88, 12-23.	1.8	31
18	Nuclear Data Uncertainty Propagation in Depletion Calculations Using Cross Section Uncertainties in One-group or Multi-group. Nuclear Data Sheets, 2015, 123, 79-83.	2.2	2

#	ARTICLE	IF	CITATIONS
19	Inventory calculation and nuclear data uncertainty propagation on light water reactor fuel using ALEPH-2 and SCALE 6.2. <i>Annals of Nuclear Energy</i> , 2015, 83, 137-146.	1.8	11
20	Comparison of nuclear data uncertainty propagation methodologies for PWR burn-up simulations. <i>Annals of Nuclear Energy</i> , 2015, 77, 101-114.	1.8	41
21	PROPAGATION OF NUCLEAR DATA UNCERTAINTIES FOR PWR CORE ANALYSIS. <i>Nuclear Engineering and Technology</i> , 2014, 46, 299-312.	2.3	25
22	Fission yield covariance generation and uncertainty propagation through fission pulse decay heat calculation. <i>Annals of Nuclear Energy</i> , 2014, 69, 331-343.	1.8	24
23	The Activities of the European Consortium on Nuclear Data Development and Analysis for Fusion. <i>Nuclear Data Sheets</i> , 2014, 120, 226-229.	2.2	5
24	Optimization of multidimensional cross-section tables for few-group core calculations. <i>Annals of Nuclear Energy</i> , 2014, 69, 226-237.	1.8	13
25	Monte Carlo uncertainty propagation approaches in ADS burn-up calculations. <i>Annals of Nuclear Energy</i> , 2013, 54, 27-35.	1.8	10
26	Methodologies for an improved prediction of the isotopic content in high burnup samples. Application to Vandellós-III reactor core. <i>Annals of Nuclear Energy</i> , 2013, 57, 199-208.	1.8	1
27	Review of the $\text{natC}(n, \hat{f}^3)$ cross section and criticality calculations of the graphite moderated reactor BR1. <i>Annals of Nuclear Energy</i> , 2013, 60, 210-217.	1.8	5
28	Uncertainty Analysis in Reactor Physics Modeling. <i>Science and Technology of Nuclear Installations</i> , 2013, 2013, 1-2.	0.8	4
29	Study on the response of IFMIF fission chambers to mixed neutron-gamma fields: PH-2 experimental tests. <i>Fusion Engineering and Design</i> , 2011, 86, 1232-1235.	1.9	8
30	Impact of activation cross-section uncertainties on the tritium production in the HFTM specimen cells. <i>Journal of Nuclear Materials</i> , 2011, 417, 1307-1310.	2.7	1
31	Nuclear data requirements for the ADS conceptual design EFIT: Uncertainty and sensitivity study. <i>Annals of Nuclear Energy</i> , 2010, 37, 1570-1579.	1.8	9
32	Feasibility of fission chambers as a neutron diagnostic in the IFMIF Test Cell. <i>Fusion Engineering and Design</i> , 2009, 84, 1570-1574.	1.9	9
33	Propagation of statistical and nuclear data uncertainties in Monte Carlo burn-up calculations. <i>Annals of Nuclear Energy</i> , 2008, 35, 714-730.	1.8	63
34	Effect of activation cross-section uncertainties on the radiological assessment of the MFE/DEMO first wall. <i>Fusion Engineering and Design</i> , 2006, 81, 1561-1565.	1.9	3
35	Methods and Results for the MSLB NEA Benchmark Using SIMTRAN and RELAP-5. <i>Nuclear Technology</i> , 2004, 146, 29-40.	1.2	10
36	Analytic Coarse-Mesh Finite-Difference Method Generalized for Heterogeneous Multidimensional Two-Group Diffusion Calculations. <i>Nuclear Science and Engineering</i> , 2003, 144, 23-35.	1.1	8

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37	Monte Carlo Uncertainty Analyses of Pulsed Activation in the National Ignition Facility Gunita Shielding. Fusion Science and Technology, 2003, 43, 473-477.	1.1	7
38	Pulsed activation of structural materials in IFE chambers. Fusion Engineering and Design, 2002, 60, 45-53.	1.9	2
39	Methods and Performance of the Three-Dimensional Pressurized Water Reactor Core Dynamics SIMTRAN On-Line Code. Nuclear Science and Engineering, 1996, 124, 111-124.	1.1	5