Hyung Jin Shim

List of Publications by Year
in descending order

33 papers	482 citations	759233 12 h-index	677142 22 g-index
33 all docs	33 docs citations	33 times ranked	176 citing authors

#	Article	IF	Citations
1	MCCARD: MONTE CARLO CODE FOR ADVANCED REACTOR DESIGN AND ANALYSIS. Nuclear Engineering and Technology, 2012, 44, 161-176.	2.3	110
2	Adjoint Sensitivity and Uncertainty Analyses in Monte Carlo Forward Calculations. Journal of Nuclear Science and Technology, 2011, 48, 1453-1461.	1.3	55
3	Generation of Few-Group Diffusion Theory Constants by Monte Carlo Code McCARD. Nuclear Science and Engineering, 2012, 172, 66-77.	1.1	32
4	Whole Core Transport Calculation Employing Hexagonal Modular Ray Tracing and CMFD Formulation. Journal of Nuclear Science and Technology, 2008, 45, 740-751.	1.3	30
5	Uncertainty Propagation in Monte Carlo Depletion Analysis. Nuclear Science and Engineering, 2011, 167, 196-208.	1.1	27
6	Real Variance Estimation Using an Intercycle Fission Source Correlation for Monte Carlo Eigenvalue Calculations. Nuclear Science and Engineering, 2009, 162, 98-108.	1.1	23
7	Dynamic Monte Carlo transient analysis for the Organization for Economic Co-operation and Development Nuclear Energy Agency (OECD/NEA) C5G7-TD benchmark. Nuclear Engineering and Technology, 2017, 49, 920-927.	2.3	21
8	Stopping Criteria of Inactive Cycle Monte Carlo Calculations. Nuclear Science and Engineering, 2007, 157, 132-141.	1.1	18
9	Memory-efficient calculations of adjoint-weighted tallies by the Monte Carlo Wielandt method. Annals of Nuclear Energy, 2016, 96, 287-294.	1.8	14
10	Optimization layer by layer networks for in-core fuel management optimization computations in PWRs. Annals of Nuclear Energy, 2001, 28, 1115-1132.	1.8	13
11	Tally efficiency analysis for Monte Carlo Wielandt method. Annals of Nuclear Energy, 2009, 36, 1694-1701.	1.8	12
12	Decay heat analysis of VHTR cores by Monte Carlo core depletion calculation. Annals of Nuclear Energy, 2010, 37, 1356-1368.	1.8	12
13	Real variance analysis of Monte Carlo eigenvalue calculation by McCARD for BEAVRS benchmark. Annals of Nuclear Energy, 2016, 90, 205-211.	1.8	12
14	Development of Generalized Perturbation Theory Algorithms for Monte Carlo Eigenvalue Calculations. Nuclear Science and Engineering, 2018, 189, 171-187.	1.1	11
15	Parallel Computing Adaptive Simulated Annealing Scheme for Fuel Assembly Loading Pattern Optimization in PWRs. Nuclear Technology, 2001, 135, 39-50.	1.2	10
16	Real Variance Estimation by Grouping Histories in Monte Carlo Eigenvalue Calculations. Nuclear Science and Engineering, 2014, 176, 58-68.	1.1	10
17	Monte Carlo burnup and its uncertainty propagation analyses for VERA depletion benchmarks by McCARD. Nuclear Engineering and Technology, 2018, 50, 1043-1050.	2.3	9
18	Uncertainty Propagation Analysis for PWR Burnup Pin-Cell Benchmark by Monte Carlo Code McCARD. Science and Technology of Nuclear Installations, 2012, 2012, 1-6.	0.8	8

#	Article	IF	Citations
19	Effect of Cross Section Uncertainties on Criticality Benchmark Problem Analysis by MCCARD. Journal of the Korean Physical Society, 2011, 59, 1252-1255.	0.7	8
20	Monte Carlo Alpha Iteration Algorithm for a Subcritical System Analysis. Science and Technology of Nuclear Installations, 2015, 2015, 1-7.	0.8	7
21	Uncertainty Quantification of Few-Group Diffusion Theory Constants Generated by the B1Theory-Augmented Monte Carlo Method. Nuclear Science and Engineering, 2013, 175, 28-43.	1.1	6
22	Monte Carlo Fuel Temperature Coefficient Estimation by an Adjoint-Weighted Correlated Sampling Method. Nuclear Science and Engineering, 2014, 177, 184-192.	1.1	6
23	McCARD for neutronics design and analysis of research reactor cores. Annals of Nuclear Energy, 2015, 82, 48-53.	1.8	6
24	UNCERTAINTY PROPAGATION ANALYSIS FOR YONGGWANG NUCLEAR UNIT 4 BY MCCARD/MASTER CORE ANALYSIS SYSTEM. Nuclear Engineering and Technology, 2014, 46, 291-298.	2.3	5
25	Monte Carlo Perturbation Analysis of Isothermal Temperature Reactivity Coefficient in Kyoto University Critical Assembly. Nuclear Technology, 2015, 191, 174-184.	1.2	4
26	Monte Carlo Few-Group Constant Generation for CANDU 6 Core Analysis. Science and Technology of Nuclear Installations, 2015, 2015, 1-11.	0.8	4
27	Monte Carlo Sensitivity and Uncertainty Analysis with Continuous-Energy Covariance Data. Nuclear Science and Engineering, 2017, 187, 154-165.	1.1	3
28	Sensitivity and uncertainty analysis of nuclear reactor reactivity coefficients by Monte Carlo second-order perturbation method. Annals of Nuclear Energy, 2018, 121, 68-76.	1.8	3
29	Determination of an Effective Detector Position for Pulsed-Neutron-Source Alpha Measurement by Time-Dependent Monte Carlo Neutron Transport Simulations. Science and Technology of Nuclear Installations, 2018, 2018, 1-7.	0.8	2
30	Estimation of kinetics parameters by Monte Carlo fixed-source calculations for point kinetic analysis of accelerator-driven system. Journal of Nuclear Science and Technology, 2020, 57, 177-186.	1.3	1
31	CAD-Based Monte Carlo Neutron Transport KSTAR Analysis for KSTAR. EPJ Web of Conferences, 2017, 153, 06019.	0.3	0
32	Stochastic Perturbation Algorithms for Kinetic Monte Carlo Simulations. , 2014, , .		0
33	McCARD for Neutronics Design and Analysis of Research Reactor Cores. , 2014, , .		0