Mohammad Reza Nematollahi

List of Publications by Year
in descending order

#	Article	IF	Citations
1	Using MCNP for success criteria evaluation of reactor trip function in probabilistic modeling of ATWS upon LOOP. Progress in Nuclear Energy, 2018, 106, 270-277.	2.9	2
2	Neutronic-thermal hydraulic coupling analysis of the fuel channel ofÂaÂnew generation of the small modular pressurized water reactor including hexagonal and square fuel assemblies using MCNP and CFX. Progress in Nuclear Energy, 2017, 98, 213-227.	2.9	14
3	Source substitution and optimization of CBXTM PGNAA analyzer in Kangan cement plant. Progress in Nuclear Energy, 2016, 90, 204-211.	2.9	6
4	Numerical study of mass transfer coefficient in a T-junction. International Journal of Hydrogen Energy, 2016, 41, 7027-7035.	7.1	2
5	Assessment of thermal hydraulics parameters of the VVER-1000 during transient conditions. International Journal of Hydrogen Energy, 2016, 41, 7103-7111.	7.1	6
6	CFX study of flow accelerated corrosion via mass transfer coefficient calculation in a double elbow. International Journal of Hydrogen Energy, 2016, 41, 7036-7046.	7.1	5
7	Moderation and shielding optimization for a 252Cf based prompt gamma neutron activation analyzer system. International Journal of Hydrogen Energy, 2016, 41, 7221-7226.	7.1	19
8	Dose assessment of radionuclides dispersion from Bushehr nuclear power plant stack under normal operation and accident conditions. International Journal of Hydrogen Energy, 2015, 40, 15198-15205.	7.1	23
9	Experimental and numerical void fraction measurement for modeled two-phase flow inside a vertical pipe. Annals of Nuclear Energy, 2015, 83, 188-192.	1.8	9
10	Reduction of core damage frequency via new design for emergency core cooling system inÂaÂtypical PWR. International Journal of Hydrogen Energy, 2015, 40, 15185-15191.	7.1	0
11	Void fraction measurement in modeled two-phase flow inside a vertical pipe by using polyethylene phantoms. International Journal of Hydrogen Energy, 2015, 40, 15206-15212.	7.1	18
12	Visualization experiment of complex flow field in a sphere-packed pipe by detailed PIV measurement. Fusion Engineering and Design, 2014, 89, 1251-1256.	1.9	4
13	Sensitivity analysis on the effect of software-induced common cause failure probability in the computer-based reactor trip system unavailability. Annals of Nuclear Energy, 2013, 57, 294-303.	1.8	18
14	Numerical study of single and two-phase models of water/Al2O3 nanofluid turbulent forced convection flow in VVER-1000 nuclear reactor. Annals of Nuclear Energy, 2013, 60, 287-294.	1.8	54
15	Numerical Simulation of Water-Based Alumina Nanofluid in Subchannel Geometry. Science and Technology of Nuclear Installations, 2012, 2012, 1-12.	0.8	32
16	Experimental Evaluation of Natural Circulation Pressure Drop in a Boiling Channel. Fusion Science and Technology, 2012, 61, 174-177.	1.1	1
17	Quantitative Evaluation of Heat Transfer in Bubble Collapse Process in Subcooled Flow Boiling. Fusion Science and Technology, 2012, 61, 186-192.	1.1	0
18	Performance evaluating of the AP1000 passive safety systems for mitigation of small break loss of coolant accident using risk assessment tool-II software. Nuclear Engineering and Design, 2012, 253, 32-40.	1.7	9

#	Article	IF	Citations
19	Comparison of T-junction flow pattern of water and sodium for different geometries of power plant piping systems. Annals of Nuclear Energy, 2012, 39, 83-93.	1.8	6
20	Simulation of a control rod ejection accident in a VVER-1000/V446 using RELAP5/Mod3.2. Annals of Nuclear Energy, 2012, 45, 106-114.	1.8	34
21	Numerical Analysis of Water-Based Nanofluid Coolant for Small Modular Reactor. , 2011, , .		4
22	Evaluating the core damage frequency of a TRIGA research reactor using risk assessment tool software. Nuclear Engineering and Design, 2011, 241, 2942-2947.	1.7	7
23	Coupling CFAST fire modeling and SAPHIRE probabilistic assessment software for internal fire safety evaluation of a typical TRIGA research reactor. Reliability Engineering and System Safety, 2010, 95, 166-172.	8.9	12
24	Development and application of a Risk Assessment Tool. Reliability Engineering and System Safety, 2008, 93, 1130-1137.	8.9	22
25	Enhancement of heat transfer in a typical pressurized water reactor by different mixing vanes on spacer grids. Energy Conversion and Management, 2008, 49, 1981-1988.	9.2	28
26	A simulation of a steam generator tube rupture in a VVER-1000 plant. Energy Conversion and Management, 2008, 49, 1972-1980.	9.2	17
27	Vibration Mechanisms of a Heated Rod Due to Subcooled Boiling Flow. , 2006, , 939.		1
28	Comparison of Flow-Induced Vibrations Versus Subcooled Boiling-Induced Vibrations on a Heated Rod in Axial Flow. , 2006, , 947.		0
29	NUCLEAR FUEL DEPLETION ANALYSIS USING MATLAB SOFTWARE. International Journal of Modern Physics C, 2006, 17, 805-815.	1.7	3
30	Vibration Characteristic of Heated Rod Induced by Subcooled Flow Boiling. Journal of Nuclear Science and Technology, 1999, 36, 575-583.	1.3	14