

# Mohammad Reza Nematollahi

## List of Publications by Year in descending order

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30  
papers

370  
citations

759233

12  
h-index

794594

19  
g-index

30  
all docs

30  
docs citations

30  
times ranked

309  
citing authors

#	ARTICLE	IF	CITATIONS
1	Numerical study of single and two-phase models of water/Al <sub>2</sub> O <sub>3</sub> nanofluid turbulent forced convection flow in VVER-1000 nuclear reactor. <i>Annals of Nuclear Energy</i> , 2013, 60, 287-294.	1.8	54
2	Simulation of a control rod ejection accident in a VVER-1000/V446 using RELAP5/Mod3.2. <i>Annals of Nuclear Energy</i> , 2012, 45, 106-114.	1.8	34
3	Numerical Simulation of Water-Based Alumina Nanofluid in Subchannel Geometry. <i>Science and Technology of Nuclear Installations</i> , 2012, 2012, 1-12.	0.8	32
4	Enhancement of heat transfer in a typical pressurized water reactor by different mixing vanes on spacer grids. <i>Energy Conversion and Management</i> , 2008, 49, 1981-1988.	9.2	28
5	Dose assessment of radionuclides dispersion from Bushehr nuclear power plant stack under normal operation and accident conditions. <i>International Journal of Hydrogen Energy</i> , 2015, 40, 15198-15205.	7.1	23
6	Development and application of a Risk Assessment Tool. <i>Reliability Engineering and System Safety</i> , 2008, 93, 1130-1137.	8.9	22
7	Moderation and shielding optimization for a <sup>252</sup> Cf based prompt gamma neutron activation analyzer system. <i>International Journal of Hydrogen Energy</i> , 2016, 41, 7221-7226.	7.1	19
8	Sensitivity analysis on the effect of software-induced common cause failure probability in the computer-based reactor trip system unavailability. <i>Annals of Nuclear Energy</i> , 2013, 57, 294-303.	1.8	18
9	Void fraction measurement in modeled two-phase flow inside a vertical pipe by using polyethylene phantoms. <i>International Journal of Hydrogen Energy</i> , 2015, 40, 15206-15212.	7.1	18
10	A simulation of a steam generator tube rupture in a VVER-1000 plant. <i>Energy Conversion and Management</i> , 2008, 49, 1972-1980.	9.2	17
11	Vibration Characteristic of Heated Rod Induced by Subcooled Flow Boiling. <i>Journal of Nuclear Science and Technology</i> , 1999, 36, 575-583.	1.3	14
12	Neutronic-thermal hydraulic coupling analysis of the fuel channel of a new generation of the small modular pressurized water reactor including hexagonal and square fuel assemblies using MCNP and CFX. <i>Progress in Nuclear Energy</i> , 2017, 98, 213-227.	2.9	14
13	Coupling CFAST fire modeling and SAPHIRE probabilistic assessment software for internal fire safety evaluation of a typical TRIGA research reactor. <i>Reliability Engineering and System Safety</i> , 2010, 95, 166-172.	8.9	12
14	Performance evaluating of the AP1000 passive safety systems for mitigation of small break loss of coolant accident using risk assessment tool-II software. <i>Nuclear Engineering and Design</i> , 2012, 253, 32-40.	1.7	9
15	Experimental and numerical void fraction measurement for modeled two-phase flow inside a vertical pipe. <i>Annals of Nuclear Energy</i> , 2015, 83, 188-192.	1.8	9
16	Evaluating the core damage frequency of a TRIGA research reactor using risk assessment tool software. <i>Nuclear Engineering and Design</i> , 2011, 241, 2942-2947.	1.7	7
17	Comparison of T-junction flow pattern of water and sodium for different geometries of power plant piping systems. <i>Annals of Nuclear Energy</i> , 2012, 39, 83-93.	1.8	6
18	Source substitution and optimization of CBXTM PGNAAL analyzer in Kangan cement plant. <i>Progress in Nuclear Energy</i> , 2016, 90, 204-211.	2.9	6

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19	Assessment of thermal hydraulics parameters of the VVER-1000 during transient conditions. International Journal of Hydrogen Energy, 2016, 41, 7103-7111.	7.1	6
20	CFX study of flow accelerated corrosion via mass transfer coefficient calculation in a double elbow. International Journal of Hydrogen Energy, 2016, 41, 7036-7046.	7.1	5
21	Numerical Analysis of Water-Based Nanofluid Coolant for Small Modular Reactor. , 2011, , .		4
22	Visualization experiment of complex flow field in a sphere-packed pipe by detailed PIV measurement. Fusion Engineering and Design, 2014, 89, 1251-1256.	1.9	4
23	NUCLEAR FUEL DEPLETION ANALYSIS USING MATLAB SOFTWARE. International Journal of Modern Physics C, 2006, 17, 805-815.	1.7	3
24	Numerical study of mass transfer coefficient in a T-junction. International Journal of Hydrogen Energy, 2016, 41, 7027-7035.	7.1	2
25	Using MCNP for success criteria evaluation of reactor trip function in probabilistic modeling of ATWS upon LOOP. Progress in Nuclear Energy, 2018, 106, 270-277.	2.9	2
26	Vibration Mechanisms of a Heated Rod Due to Subcooled Boiling Flow. , 2006, , 939.		1
27	Experimental Evaluation of Natural Circulation Pressure Drop in a Boiling Channel. Fusion Science and Technology, 2012, 61, 174-177.	1.1	1
28	Comparison of Flow-Induced Vibrations Versus Subcooled Boiling-Induced Vibrations on a Heated Rod in Axial Flow. , 2006, , 947.		0
29	Quantitative Evaluation of Heat Transfer in Bubble Collapse Process in Subcooled Flow Boiling. Fusion Science and Technology, 2012, 61, 186-192.	1.1	0
30	Reduction of core damage frequency via new design for emergency core cooling system in Atypical PWR. International Journal of Hydrogen Energy, 2015, 40, 15185-15191.	7.1	0