List of Publications by Year in descending order

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| # | Article | IF | CITATIONS |
|----|---|-----|-----------|
| 1 | A Coaxial Helicity Injection System for Nonsolenoidal Startup Studies on the PEGASUS-III Experiment. IEEE Transactions on Plasma Science, 2022, 50, 4015-4020. | 1.3 | 3 |
| 2 | Initial Results from High-Field-Side Transient CHI Start-Up on QUEST. Plasma and Fusion Research, 2021, 16, 2402048-2402048. | 0.7 | 2 |
| 3 | 3D radiation, density, and MHD structures following neon shattered pellet injection into stable DIII-D Super H-mode discharges. Nuclear Fusion, 2021, 61, 066040. | 3.5 | 7 |
| 4 | Modeling of carbon pellets disruption mitigation in an NSTX-U plasma. Nuclear Fusion, 2021, 61, 116003. | 3.5 | 3 |
| 5 | Prototype tests of the electromagnetic particle injector-2 for fast time response disruption mitigation in tokamaks. Nuclear Fusion, 2021, 61, 126034. | 3.5 | 8 |
| 6 | Active Radiative Liquid Lithium Divertor for Handling Transient High Heat Flux Events. Journal of Fusion Energy, 2020, 39, 402-410. | 1.2 | 3 |
| 7 | Shattered pellet penetration in low and high energy plasmas on DIII-D. Nuclear Fusion, 2020, 60, 036014. | 3.5 | 14 |
| 8 | Modeling of Ablatant Deposition from Electromagnetically Driven Radiative Pellets for Disruption Mitigation Studies. Fusion Science and Technology, 2019, 75, 767-774. | 1.1 | 2 |
| 9 | Particle balance investigation with the combination of the hydrogen barrier model and rate equations of hydrogen state in long duration discharges on an all-metal plasma facing wall in QUEST. Nuclear Fusion, 2019, 59, 076007. | 3.5 | 11 |
| 10 | Estimation of fuel particle balance in steady state operation with hydrogen barrier model. Nuclear Materials and Energy, 2019, 19, 544-549. | 1.3 | 5 |
| 11 | NSTX/NSTX-U theory, modeling and analysis results. Nuclear Fusion, 2019, 59, 112007. | 3.5 | 20 |
| 12 | Application of transient CHI plasma startup to future ST and AT devices. Physics of Plasmas, 2019, 26, 032501. | 1.9 | 2 |
| 13 | Electromagnetic particle injector for fast time response disruption mitigation in tokamaks. Nuclear Fusion, 2019, 59, 016021. | 3.5 | 14 |
| 14 | Supersonic Gas Injector for Plasma Fueling in the National Spherical Torus Experiment. Fusion Science and Technology, 2019, 75, 1-17. | 1.1 | 6 |
| 15 | Application of Townsend avalanche theory to tokamak startup by coaxial helicity injection. Nuclear Fusion, 2018, 58, 016013. | 3.5 | 17 |
| 16 | Scenario development during commissioning operations on the National Spherical Torus Experiment Upgrade. Nuclear Fusion, 2018, 58, 046010. | 3.5 | 25 |
| 17 | Initial results from solenoid-free plasma start-up using Transient CHI on QUEST. Plasma Physics and Controlled Fusion, 2018, 60, 115001. | 2.1 | 15 |
| 18 | TSC Simulation of Transient CHI in New Electrode Configuration on QUEST. Plasma and Fusion Research, 2018, 13, 3402059-3402059. | 0.7 | 0 |

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| 19 | Overview of NSTX Upgrade initial results and modelling highlights. Nuclear Fusion, 2017, 57, 102006. | 3.5 | 45 |
| 20 | Current Start-Up Using the New CHI System. Plasma and Fusion Research, 2017, 12, 1202020-1202020. | 0.7 | 11 |
| 21 | Time-Dependent Simulations of Fast-Wave Heated High-Non-Inductive-Fraction H-Mode Plasmas in the National Spherical Torus Experiment Upgrade. EPJ Web of Conferences, 2017, 157, 03052. | 0.3 | 0 |
| 22 | Snowflake Divertor Experiments in the DIII-D, NSTX, and NSTX-U Tokamaks Aimed at the Development of the Divertor Power Exhaust Solution. IEEE Transactions on Plasma Science, 2016, 44, 3445-3455. | 1.3 | 14 |
| 23 | Simplifying the ST and AT Concepts. Journal of Fusion Energy, 2016, 35, 34-40. | 1.2 | 4 |
| 24 | Fusion nuclear science facilities and pilot plants based on the spherical tokamak. Nuclear Fusion, 2016, 56, 106023. | 3.5 | 119 |
| 25 | Power Balance Estimation in Long Duration Discharges on QUEST. Plasma Science and Technology, 2016, 18, 1069-1075. | 1.5 | 19 |
| 26 | Massive Gas Injection Valve Development for NSTX-U. IEEE Transactions on Plasma Science, 2016, 44, 1547-1552. | 1.3 | 6 |
| 27 | Large-volume flux closure during plasmoid-mediated reconnection in coaxial helicity injection. Nuclear Fusion, 2016, 56, 044002. | 3.5 | 17 |
| 28 | Reconstruction of NSTX midplane neutral density profiles from visible imaging data. Journal of Nuclear Materials, 2015, 463, 897-901. | 2.7 | 5 |
| 29 | Development of fully non-inductive plasmas heated by medium and high-harmonic fast waves in the national spherical torus experiment upgrade. AIP Conference Proceedings, 2015, , . | 0.4 | 0 |
| 30 | Design Description for a Coaxial Helicity Injection Plasma Start-Up System for a ST-FNSF. Fusion Science and Technology, 2015, 68, 674-679. | 1.1 | 11 |
| 31 | Fast Time Response Electromagnetic Disruption Mitigation Concept. Fusion Science and Technology, 2015, 68, 797-805. | 1.1 | 5 |
| 32 | A megawatt-level 28 GHz heating system for the National Spherical Torus Experiment Upgrade. EPJ Web of Conferences, 2015, 87, 02013. | 0.3 | 5 |
| 33 | Plasmoids Formation During Simulations of Coaxial Helicity Injection in the National Spherical Torus Experiment. Physical Review Letters, 2015, 114, 205003. | 7.8 | 46 |
| 34 | An overview of recent physics results from NSTX. Nuclear Fusion, 2015, 55, 104002. | 3.5 | 21 |
| 35 | The role of MHD in 3D aspects of massive gas injection. Nuclear Fusion, 2015, 55, 073032. | 3.5 | 28 |
| 36 | Progress toward commissioning and plasma operation in NSTX-U. Nuclear Fusion, 2015, 55, 073007. | 3.5 | 16 |

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| 37 | Physics of forced magnetic reconnection in coaxial helicity injection experiments in National Spherical Torus Experiment. Physics of Plasmas, 2014, 21, . | 1.9 | 11 |
| 38 | Design and operation of a fast electromagnetic inductive massive gas injection valve for NSTX-U. Review of Scientific Instruments, 2014, 85, 11E801. | 1.3 | 9 |
| 39 | Physics design of a 28 GHz electron heating system for the National Spherical Torus experiment upgrade. , 2014, , . | | 0 |
| 40 | Solenoid-free plasma start-up in spherical tokamaks. Plasma Physics and Controlled Fusion, 2014, 56, 103001. | 2.1 | 26 |
| 41 | Design Details of the Transient CHI Plasma Start-up System on NSTX-U. IEEE Transactions on Plasma Science, 2014, 42, 2154-2160. | 1.3 | 3 |
| 42 | Advanced divertor configurations with large flux expansion. Journal of Nuclear Materials, 2013, 438, S96-S101. | 2.7 | 24 |
| 43 | Overview of physics results from the conclusive operation of the National Spherical Torus Experiment. Nuclear Fusion, 2013, 53, 104007. | 3.5 | 53 |
| 44 | Design description of the coaxial helicity injection (CHI) system on NSTX-U. , 2013, , . | | 0 |
| 45 | Characterization of fueling NSTX H-mode plasmas diverted to a liquid lithium divertor. Journal of Nuclear Materials, 2013, 438, S488-S492. | 2.7 | 8 |
| 46 | Non-inductive plasma start-up on NSTX and projections to NSTX-U using transient CHI. Nuclear Fusion, 2013, 53, 073017. | 3.5 | 28 |
| 47 | Magnetic reconnection process in transient coaxial helicity injection. Physics of Plasmas, 2013, 20, . | 1.9 | 23 |
| 48 | Resistive magnetohydrodynamic simulations of helicity-injected startup plasmas in National Spherical Torus eXperiment. Physics of Plasmas, 2013, 20, . | 1.9 | 13 |
| 49 | Recent progress in the NSTX/NSTX-U lithium programme and prospects for reactor-relevant liquid-lithium based divertor development. Nuclear Fusion, 2013, 53, 113030. | 3.5 | 32 |
| 50 | Modification of the NSTX-U outboard and Inboard Divertor tiles for the protection of the PF-1C coils. , 2013, , . | | 0 |
| 51 | Snowflake divertor configuration studies in National Spherical Torus Experiment. Physics of Plasmas, 2012, 19, . | 1.9 | 67 |
| 52 | Diagnostic options for radiative divertor feedback control on NSTX-U. Review of Scientific Instruments, 2012, 83, 10D716. | 1.3 | 2 |
| 53 | Overview of the physics and engineering design of NSTX upgrade. Nuclear Fusion, 2012, 52, 083015. | 3.5 | 177 |
| 54 | NSTX plasma operation with a Liquid Lithium Divertor. Fusion Engineering and Design, 2012, 87, 1724-1731. | 1.9 | 72 |

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| 55 | Recent progress of NSTX lithium program and opportunities for magnetic fusion research. Fusion Engineering and Design, 2012, 87, 1770-1776. | 1.9 | 11 |
| 56 | The effect of progressively increasing lithium coatings on plasma discharge characteristics, transport, edge profiles and ELM stability in the National Spherical Torus Experiment. Nuclear Fusion, 2012, 52, 083001. | 3.5 | 101 |
| 57 | Massive Gas Injection Plans for Disruption Mitigation Studies in NSTX-U. IEEJ Transactions on Fundamentals and Materials, 2012, 132, 468-471. | 0.2 | 0 |
| 58 | Transient Coaxial Helicity Injection Plasma Start-up in NSTX and CHI Program Plans on NSTX-U. IEEJ Transactions on Fundamentals and Materials, 2012, 132, 462-467. | 0.2 | 0 |
| 59 | Overview of physics results from NSTX. Nuclear Fusion, 2011, 51, 094011. | 3.5 | 10 |
| 60 | "Snowflake―divertor configuration in NSTX. Journal of Nuclear Materials, 2011, 415, S365-S368. | 2.7 | 26 |
| 61 | Reduction of low-Z impurities during plasma start-up. Journal of Nuclear Materials, 2011, 415, S1017-S1020. | 2.7 | 3 |
| 62 | NSTX plasma response to lithium coated divertor. Journal of Nuclear Materials, 2011, 415, S400-S404. | 2.7 | 32 |
| 63 | Taming the plasma–material interface with the â€~snowflake' divertor in NSTX. Nuclear Fusion, 2011, 51, 012001. | 3.5 | 73 |
| 64 | Transient CHI start-up simulations with the TSC. Nuclear Fusion, 2011, 51, 113018. | 3.5 | 12 |
| 65 | Demonstration of 300 kA CHI-startup current, coupling to transformer drive and flux savings on NSTX. Nuclear Fusion, 2011, 51, 063008. | 3.5 | 17 |
| 66 | Experimental demonstration of tokamak inductive flux saving by transient coaxial helicity injection on national spherical torus experiment. Physics of Plasmas, 2011, 18, . | 1.9 | 21 |
| 67 | Lithium coatings on NSTX plasma facing components and its effects on boundary control, core plasma performance, and operation. Fusion Engineering and Design, 2010, 85, 865-873. | 1.9 | 35 |
| 68 | Demonstration of Plasma Start-up in HIT-II and NSTX Using Transient Coaxial Helicity Injection. Journal of Fusion Energy, 2010, 29, 540-542. | 1.2 | 0 |
| 69 | Implications of NSTX lithium results for magnetic fusion research. Fusion Engineering and Design, 2010, 85, 882-889. | 1.9 | 17 |
| 70 | Demonstration of Tokamak Ohmic Flux Saving by Transient Coaxial Helicity Injection in the National Spherical Torus Experiment. Physical Review Letters, 2010, 104, 095003. | 7.8 | 44 |
| 71 | Overview of L–H power threshold studies in NSTX. Nuclear Fusion, 2010, 50, 064010. | 3.5 | 40 |
| 72 | Ramp-Up of CHI-Initiated Plasmas on NSTX. IEEE Transactions on Plasma Science, 2010, 38, 371-374. | 1.3 | 6 |

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| 73 | Divertor heat flux mitigation in the National Spherical Torus Experiment. Physics of Plasmas, 2009, 16, 022501. | 1.9 | 33 |
| 74 | Ramp-up of CHI initiated plasmas on NSTX. , 2009, , . | | 0 |
| 75 | Solenoid-free plasma startup in NSTX using transient CHI. Nuclear Fusion, 2009, 49, 065006. | 3.5 | 19 |
| 76 | REVIEW OF THE NATIONAL SPHERICAL TORUS EXPERIMENT RESEARCH RESULTS. , 2009, , . | | 0 |
| 77 | TRANSIENT CHI START-UP IN NSTX. , 2009, , . | | 0 |
| 78 | NIMROD EXTENDED MHD SIMULATIONS FOR DISRUPTION MITIGATION STUDIES. , 2009, , . | | 0 |
| 79 | SOLENOID-FREE PLASMA START-UP IN HIT-II. , 2009, , . | | 0 |
| 80 | SPHEROMAK FORMATION BY STEADY INDUCTIVE HELICITY INJECTION. , 2009, , . | | 0 |
| 81 | Divertor heat flux mitigation in high-performance H-mode discharges in the National Spherical Torus Experiment. Nuclear Fusion, 2009, 49, 095025. | 3.5 | 36 |
| 82 | Evaporated lithium surface coatings in NSTX. Journal of Nuclear Materials, 2009, 390-391, 1000-1004. | 2.7 | 74 |
| 83 | Solenoid-free Plasma Start-up in NSTX using Transient CHI. Journal of Fusion Energy, 2009, 28, 200-202. | 1.2 | 2 |
| 84 | On the secular density rises in NBI-heated H-mode plasmas in NSTX. Journal of Nuclear Materials, 2009, 390-391, 516-519. | 2.7 | 10 |
| 85 | Transition to ELM-free improved H-mode by lithium deposition on NSTX graphite divertor surfaces. Journal of Nuclear Materials, 2009, 390-391, 764-767. | 2.7 | 67 |
| 86 | Overview of results from the National Spherical Torus Experiment (NSTX). Nuclear Fusion, 2009, 49, 104016. | 3.5 | 41 |
| 87 | Solenoid-Less Plasma Start-Up in NSTX Using Transient CHI. Fusion Science and Technology, 2009, 56, 512-517. | 1.1 | 1 |
| 88 | Plasma Start-up in HIT-II and NSTX Using Transient Coaxial Helicity Injection. Journal of Fusion Energy, 2008, 27, 96-99. | 1.2 | 1 |
| 89 | Advanced fuelling system for ITER. Fusion Engineering and Design, 2008, 83, 1368-1374. | 1.9 | 23 |
| 90 | Temperature and density characteristics of the Helicity Injected Torus-II spherical tokamak indicating closed flux sustainment using coaxial helicity injection. Physics of Plasmas, 2008, 15, 082501. | 1.9 | 3 |

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| 91 | The effect of lithium surface coatings on plasma performance in the National Spherical Torus Experiment. Physics of Plasmas, 2008, 15, . | 1.9 | 153 |
| 92 | Advanced Fueling System for Steady-State Operation of a Fusion Reactor. Fusion Science and Technology, 2008, 54, 71-74. | 1.1 | 1 |
| 93 | Non-inductive solenoid-less plasma current startup in NSTX using transient CHI. Nuclear Fusion, 2007, 47, 792-799. | 3.5 | 23 |
| 94 | Transient coaxial helicity injection for solenoid-free plasma startup in HIT-II. Physics of Plasmas, 2007, 14, 022504. | 1.9 | 12 |
| 95 | Transport with reversed shear in the National Spherical Torus Experiment. Physics of Plasmas, 2007, 14, 056119. | 1.9 | 37 |
| 96 | Plasma startup in the National Spherical Torus Experiment using transient coaxial helicity injection. Physics of Plasmas, 2007, 14, 056106. | 1.9 | 8 |
| 97 | NSTX Plasma Start-Up Using Transient Coaxial Helicity Injection. Fusion Science and Technology, 2007, 52, 393-397. | 1.1 | 3 |
| 98 | Progress towards steady state at low aspect ratio on the National Spherical Torus Experiment (NSTX). Nuclear Fusion, 2007, 47, 1376-1382. | 3.5 | 15 |
| 99 | Overview of recent physics results from the National Spherical Torus Experiment (NSTX). Nuclear Fusion, 2007, 47, S645-S657. | 3.5 | 40 |
| 100 | Solenoid-free Plasma Start-up in HIT-II and NSTX using Transient CHI. Journal of Fusion Energy, 2007, 26, 159-162. | 1.2 | 2 |
| 101 | Divertor heat flux reduction and detachment experiments in NSTX. Journal of Nuclear Materials, 2007, 363-365, 432-436. | 2.7 | 17 |
| 102 | Effect of lithium PFC coatings on NSTX density control. Journal of Nuclear Materials, 2007, 363-365, 791-796. | 2.7 | 54 |
| 103 | E × B Plasma Rotation and n = 1 Oscillation Observed in the NSTX-CHI Experiments. Plasma and Fusion Research, 2007, 2, 035-035. | 0.7 | 3 |
| 104 | Cross-machine comparison of resonant field amplification and resistive wall mode stabilization by plasma rotation. Physics of Plasmas, 2006, 13, 056107. | 1.9 | 100 |
| 105 | Plasma start-up using transient CHI on NSTX. , 2006, , . | | 0 |
| 106 | New capabilities and results for the National Spherical Torus Experiment. Nuclear Fusion, 2006, 46, S565-S572. | 3.5 | 28 |
| 107 | Advanced Fueling System for Use as a Burn Control Tool in a Burning Plasma Device. Fusion Science and Technology, 2006, 50, 84-88. | 1.1 | 7 |
| 108 | Progress towards steady state on NSTX. Nuclear Fusion, 2006, 46, S22-S28. | 3.5 | 17 |

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| 109 | Characterization of small, Type V edge-localized modes in the National Spherical Torus Experiment. Physics of Plasmas, 2006, 13, 092510. | 1.9 | 33 |
| 110 | Efficient Generation of Closed Magnetic Flux Surfaces in a Large Spherical Tokamak Using Coaxial Helicity Injection. Physical Review Letters, 2006, 97, 175002. | 7.8 | 45 |
| 111 | Effect of plasma shaping on performance in the National Spherical Torus Experiment. Physics of Plasmas, 2006, 13, 056122. | 1.9 | 33 |
| 112 | ELMs and the H-mode pedestal in NSTX. Journal of Nuclear Materials, 2005, 337-339, 727-731. | 2.7 | 28 |
| 113 | Development of NSTX particle control techniques. Journal of Nuclear Materials, 2005, 337-339, 495-499. | 2.7 | 11 |
| 114 | Observation of a high performance operating regime with small edge-localized modes in the National Spherical Torus Experiment. Nuclear Fusion, 2005, 45, 264-270. | 3.5 | 53 |
| 115 | H-mode pedestal, ELM and power threshold studies in NSTX. Nuclear Fusion, 2005, 45, 1066-1077. | 3.5 | 68 |
| 116 | Design, installation and performance of the new insulator for NSTX CHI experiments. , 2005, , . | | 0 |
| 117 | Non-inductive solenoid-free plasma start-up using coaxial helicity injection. Nuclear Fusion, 2005, 45, L15-L19. | 3.5 | 20 |
| 118 | Supersonic gas injector for plasma fueling. , 2005, , . | | 0 |
| 119 | Progress towards high performance plasmas in the National Spherical Torus Experiment (NSTX). Nuclear Fusion, 2005, 45, S168-S180. | 3.5 | 60 |
| 120 | Solenoid-free Plasma Startup in NSTX using Coaxial Helicity Injection. IEEJ Transactions on Fundamentals and Materials, 2005, 125, 895-901. | 0.2 | 0 |
| 121 | Status and Plans for the National Spherical Torus Experimental Research Facility. IEEJ Transactions on Fundamentals and Materials, 2005, 125, 868-880. | 0.2 | 1 |
| 122 | Next-step spherical torus experiment and spherical torus strategy in the course of development of fusion energy. Nuclear Fusion, 2004, 44, 452-463. | 3.5 | 30 |
| 123 | Effect of gas fuelling location on H-mode access in NSTX. Plasma Physics and Controlled Fusion, 2004, 46, A305-A313. | 2.1 | 33 |
| 124 | Experimental demonstration of plasma startup by coaxial helicity injection. Physics of Plasmas, 2004, 11, 2565-2572. | 1.9 | 20 |
| 125 | Fast neutral pressure gauges in NSTX. Review of Scientific Instruments, 2004, 75, 4347-4349. | 1.3 | 11 |
| 126 | Core fueling and edge particle flux analysis in ohmically and auxiliary heated NSTX plasmas. Journal of Nuclear Materials, 2003, 313-316, 573-578. | 2.7 | 14 |

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| 127 | Progress towards high-performance, steady-state spherical torus. Plasma Physics and Controlled Fusion, 2003, 45, A335-A350. | 2.1 | 25 |
| 128 | Results of NSTX heating experiments. IEEE Transactions on Plasma Science, 2003, 31, 60-67. | 1.3 | 2 |
| 129 | Fast neutral pressure measurements in NSTX. Review of Scientific Instruments, 2003, 74, 1900-1904. | 1.3 | 6 |
| 130 | Demonstration of Plasma Startup by Coaxial Helicity Injection. Physical Review Letters, 2003, 90, 075005. | 7.8 | 54 |
| 131 | H-mode threshold and dynamics in the National Spherical Torus Experiment. Physics of Plasmas, 2003, 10, 1755-1764. | 1.9 | 27 |
| 132 | Exploration of high harmonic fast wave heating on the National Spherical Torus Experiment. Physics of Plasmas, 2003, 10, 1733-1738. | 1.9 | 31 |
| 133 | The national spherical torus experiment (NSTX) research programme and progress towards high beta, long pulse operating scenarios. Nuclear Fusion, 2003, 43, 1653-1664. | 3.5 | 49 |
| 134 | Recent results from the National Spherical Torus Experiment. Plasma Physics and Controlled Fusion, 2003, 45, 657-669. | 2.1 | 23 |
| 135 | Current drive experiments in the helicity injected torus (HIT-II). Physics of Plasmas, 2002, 9, 2006-2013. | 1.9 | 34 |
| 136 | Overview of impurity control and wall conditioning in NSTX. Journal of Nuclear Materials, 2001, 290-293, 1185-1189. | 2.7 | 16 |
| 137 | Non-inductive current generation in NSTX using coaxial helicity injection. Nuclear Fusion, 2001, 41, 1081-1086. | 3.5 | 66 |
| 138 | Overview of the initial NSTX experimental results. Nuclear Fusion, 2001, 41, 1435-1447. | 3.5 | 49 |
| 139 | Current drive experiments in the HIT-II spherical tokamak. Nuclear Fusion, 2001, 41, 679-685. | 3.5 | 20 |
| 140 | Initial results from coaxial helicity injection experiments in NSTX. Plasma Physics and Controlled Fusion, 2001, 43, 305-312. | 2.1 | 16 |
| 141 | Initial physics results from the National Spherical Torus Experiment. Physics of Plasmas, 2001, 8, 1977-1987. | 1.9 | 46 |
| 142 | Exploration of spherical torus physics in the NSTX device. Nuclear Fusion, 2000, 40, 557-561. | 3.5 | 363 |
| 143 | Compact toroid fuelling for ITER. Fusion Engineering and Design, 1998, 39-40, 977-985. | 1.9 | 15 |
| 144 | Design and Operation of a Passively Switched Repetitive Compact Toroid Plasma Accelerator. Fusion Science and Technology, 1998, 33, 252-272. | 0.6 | 29 |

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| 145 | Experimental demonstration of tokamak fuelling by compact toroid injection. Nuclear Fusion, 1997, 37, 967-972. | 3.5 | 43 |
| 146 | Compact Toroid Fueling for ITER. Fusion Science and Technology, 1995, 28, 619-624. | 0.6 | 6 |
| 147 | Experimental Demonstration of Nondisruptive, Central Fueling of a Tokamak by Compact Toroid Injection. Physical Review Letters, 1994, 73, 3101-3104. | 7.8 | 90 |
| 148 | Energy balance in the CSSU device. Nuclear Fusion, 1993, 33, 1685-1694. | 3.5 | 2 |
| 149 | A simple fast pulse gas valve using a dynamic pressure differential as the primary closing mechanism. Review of Scientific Instruments, 1993, 64, 1410-1413. | 1.3 | 15 |
| 150 | Design of the Compact Toroid Fueler for Center Fueling Tokamak de Varennes. Fusion Science and Technology, 1993, 24, 239-250. | 0.6 | 18 |
| 151 | Initial results from the Coaxial Slow Source FRC device. Nuclear Fusion, 1987, 27, 1478-1488. | 3.5 | 40 |
| 152 | Making of the NSTX facility. , 0, , . | | 3 |
| 153 | NSTX high field side gas fueling system. , 0, , . | | 2 |