

Michael Corradini

List of Publications by Year in descending order

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Version: 2024-02-01

37
papers

1,020
citations

516710

16
h-index

434195

31
g-index

38
all docs

38
docs citations

38
times ranked

776
citing authors

#	ARTICLE	IF	CITATIONS
1	Modeling high-speed viscous liquid sheet atomization. International Journal of Multiphase Flow, 1999, 25, 1073-1097.	3.4	313
2	Heat transfer to water at supercritical pressures in a circular and square annular flow geometry. International Journal of Heat and Fluid Flow, 2008, 29, 156-166.	2.4	109
3	Wire-mesh sensors: A review of methods and uncertainty in multiphase flows relative to other measurement techniques. Nuclear Engineering and Design, 2018, 337, 205-220.	1.7	64
4	Evaluation of steam corrosion and water quenching behavior of zirconium-silicide coated LWR fuel claddings. Journal of Nuclear Materials, 2018, 499, 256-267.	2.7	54
5	Transient pool boiling heat transfer of oxidized and roughened Zircaloy-4 surfaces during water quenching. International Journal of Heat and Mass Transfer, 2018, 120, 435-446.	4.8	49
6	A fresh look at nuclear energy. Science, 2019, 363, 105-105.	12.6	49
7	A thermodynamically consistent and fully conservative treatment of contact discontinuities for compressible multicomponent flows. Journal of Computational Physics, 2004, 195, 528-559.	3.8	34
8	Bubble Dynamics in Pool Boiling on Nanoparticle-Coated Surfaces. Heat Transfer Engineering, 2015, 36, 1013-1027.	1.9	33
9	Nuclear Energy in a Carbon-Constrained World: Big Challenges and Big Opportunities. IEEE Power and Energy Magazine, 2019, 17, 69-77.	1.6	32
10	Measurement of supercritical CO2 critical flow: Effects of L/D and surface roughness. Nuclear Engineering and Design, 2009, 239, 949-955.	1.7	31
11	A CFD study of wave influence on film steam condensation in the presence of non-condensable gas. Nuclear Engineering and Design, 2016, 305, 303-313.	1.7	27
12	Accident tolerant clad material modeling by MELCOR: Benchmark for SURRY Short Term Station Black Out. Nuclear Engineering and Design, 2017, 313, 458-469.	1.7	26
13	Prediction of falling film evaporation on the AP1000 passive containment cooling system using ANSYS FLUENT code. Annals of Nuclear Energy, 2016, 95, 168-175.	1.8	22
14	Evaluation of critical heat flux of ATF candidate coating materials in pool boiling. Nuclear Engineering and Design, 2019, 354, 110166.	1.7	21
15	CRITICAL FLOW EXPERIMENT AND ANALYSIS FOR SUPERCRITICAL FLUID. Nuclear Engineering and Technology, 2008, 40, 133-138.	2.3	18
16	Comparison of CORA & MELCOR core degradation simulation and the MELCOR oxidation model. Nuclear Engineering and Design, 2014, 276, 191-201.	1.7	16
17	Quantification of the effect of Cr-coated-Zircaloy cladding during a short term station black out. Nuclear Engineering and Design, 2020, 363, 110678.	1.7	15
18	Analysis of KROTOS KS-2 and KS-4 steam explosion experiments with TEXAS-VI. Nuclear Engineering and Design, 2016, 309, 104-112.	1.7	12

#	ARTICLE	IF	CITATIONS
19	Revisiting Insights from Three Mile Island Unit 2 Postaccident Examinations and Evaluations in View of the Fukushima Daiichi Accident. Nuclear Science and Engineering, 2012, 172, 223-248.	1.1	11
20	Mechanistic CHF modeling for natural circulation applications in SMR. Nuclear Engineering and Design, 2016, 310, 604-611.	1.7	10
21	Pool boiling critical heat flux studies of accident tolerant fuel cladding materials. Nuclear Engineering and Design, 2020, 370, 110919.	1.7	9
22	Transient safety evaluation of the heat pipe microreactor "Potential energy source for hydrogen production. International Journal of Hydrogen Energy, 2021, 46, 38887-38902.	7.1	9
23	Boiling performance and material robustness of modified surfaces with multi scale structures for fuel cladding development. Nuclear Engineering and Design, 2015, 291, 204-211.	1.7	8
24	Flow pattern transition instabilities in a natural circulation cooling facility. Nuclear Engineering and Design, 2018, 332, 267-278.	1.7	8
25	DNB type critical heat flux prediction in rod bundles with simplified grid spacer based on Liquid Sublayer Dryout model. Nuclear Engineering and Design, 2019, 351, 94-105.	1.7	8
26	Critical Heat Flux in TRIGA-Fueled Reactors Cooled by Natural Convection. Nuclear Science and Engineering, 2012, 172, 249-258.	1.1	7
27	Stratified steam explosion energetics. Nuclear Engineering and Technology, 2019, 51, 95-103.	2.3	7
28	Long-Term Validation of the Molten Fuel-Moderator Interactions Model. Nuclear Technology, 2010, 169, 114-125.	1.2	6
29	NEW REACTOR TECHNOLOGY: SAFETY IMPROVEMENTS IN NUCLEAR POWER SYSTEMS. Health Physics, 2007, 93, 547-559.	0.5	2
30	Monitoring dry-cask storage using thermoelectric powered wireless sensors. , 2013, , .		2
31	Study of Critical Heat Flux in Natural Convection-Cooled TRIGA Reactors with Single Annulus and Rod Bundle Geometries. Nuclear Science and Engineering, 2015, 180, 141-153.	1.1	2
32	A small reactor design for ⁹⁹ Mo production with novel fuel. Journal of Radioanalytical and Nuclear Chemistry, 2015, 305, 23-30.	1.5	2
33	An analysis of air-water flow phenomena due to a pipe break under sub-atmospheric pressures using TRACE. Nuclear Engineering and Design, 2021, 374, 111064.	1.7	2
34	Solid particle effects on heat transfer in a multi-layered molten pool with gas injection. Nuclear Engineering and Design, 2006, 236, 2245-2263.	1.7	1
35	Modeling Molten Fuel-Moderator Interactions for the CANDU Flow Blockage Accident. Nuclear Technology, 2010, 169, 97-113.	1.2	1
36	Transient Two-Dimensional Hydrodynamic Experiments. Nuclear Science and Engineering, 2010, 165, 180-199.	1.1	0

#	ARTICLE	IF	CITATIONS
37	Thermal Conductivity Measurement of Granular $\text{UO}_2(\text{NO}_3)_2 \cdot 6\text{H}_2\text{O}$. Nuclear Technology, 2017, 197, 191-200.	1.2	0