

# Alessandro Del Nevo

## List of Publications by Year in descending order

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142  
papers

2,313  
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144  
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docs citations

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times ranked

761  
citing authors

| #  | ARTICLE   | IF  | CITATIONS |
|----|---|-----|-----------|
| 1  | Tokamak cooling systems and power conversion system options. Fusion Engineering and Design, 2022, 178, 113093.  | 1.9 | 26        |
| 2  | Maturation of critical technologies for the DEMO balance of plant systems. Fusion Engineering and Design, 2022, 179, 113096.  | 1.9 | 24        |
| 3  | Status of maturation of critical technologies and systems design: Breeding blanket. Fusion Engineering and Design, 2022, 179, 113116.   | 1.9 | 44        |
| 4  | Assessment of SIMMER-III code in predicting Water Cooled Lithium Lead Breeding Blanket in-box-Loss of Coolant Accident. Fusion Engineering and Design, 2021, 163, 112127.                           | 1.9 | 13        |
| 5  | Experimental Investigation on CIRCE-HERO for the EU DEMO PbLi/Water Heat Exchanger Development. Energies, 2021, 14, 628.  | 3.1 | 3         |
| 6  | Analysis of the thermal-hydraulic behavior of the EU-DEMO WCLL breeding blanket cooling systems during a loss of flow accident. Fusion Engineering and Design, 2021, 164, 112206.                   | 1.9 | 10        |
| 7  | Study of the EU-DEMO WCLL Breeding Blanket Primary Cooling Circuits Thermal-Hydraulic Performances during Transients Belonging to LOFA Category. Energies, 2021, 14, 1541.                          | 3.1 | 11        |
| 8  | Post-test analysis of Series D experiments in LIFUS5/Mod3 facility for SIMMER code validation of WCLL-BB In-box LOCA. Fusion Engineering and Design, 2021, 165, 112268.                             | 1.9 | 6         |
| 9  | Electromagnetic analysis activities in support of the Breeding Blanket during the DEMO Pre-Conceptual Design Phase: Methodology and main results. Fusion Engineering and Design, 2021, 166, 112285. | 1.9 | 10        |
| 10 | Total loss of flow benchmark in CIRCE-HERO integral test facility. Nuclear Engineering and Design, 2021, 376, 111086.   | 1.7 | 11        |
| 11 | Conceptual design of the main Ancillary Systems of the ITER Water Cooled Lithium Lead Test Blanket System. Fusion Engineering and Design, 2021, 167, 112345.  | 1.9 | 11        |
| 12 | Structural assessment of the EU-DEMO WCLL Central Outboard Blanket segment under normal and off-normal operating conditions. Fusion Engineering and Design, 2021, 167, 112350.                      | 1.9 | 10        |
| 13 | Thermo-hydraulic analysis of PbLi ancillary system of WCLL TBM undergoing in-box LOCA. Fusion Engineering and Design, 2021, 168, 112614.  | 1.9 | 5         |
| 14 | Nuclear performances of the water-cooled lithium lead DEMO reactor: Neutronic analysis on a fully heterogeneous model. Fusion Engineering and Design, 2021, 168, 112514.                            | 1.9 | 20        |
| 15 | Development of a PbLi heat exchanger for EU DEMO fusion reactor: Experimental test and system code assessment. Fusion Engineering and Design, 2021, 169, 112462.                                    | 1.9 | 1         |
| 16 | Preliminary analysis of an in-box LOCA in the breeding unit of the WCLL TBM for the ITER reactor with SIMMER-IV code. Fusion Engineering and Design, 2021, 169, 112472.                             | 1.9 | 8         |
| 17 | Overview on Lead-Cooled Fast Reactor Design and Related Technologies Development in ENEA. Energies, 2021, 14, 5157.   | 3.1 | 25        |
| 18 | Pre-conceptual design of EU DEMO balance of plant systems: Objectives and challenges. Fusion Engineering and Design, 2021, 169, 112504.   | 1.9 | 25        |

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| 19 | Preliminary design of a helical coil steam generator mock-up for the CIRCE facility for the development of DEMO LiPb heat exchanger. Fusion Engineering and Design, 2021, 169, 112459.                   | 1.9 | 8         |
| 20 | Modelling of thermal conductivity and melting behaviour of minor actinide-MOX fuels and assessment against experimental and molecular dynamics data. Journal of Nuclear Materials, 2021, 557, 153312.    | 2.7 | 12        |
| 21 | Assessment of three European fuel performance codes against the SUPERFACT-1 fast reactor irradiation experiment. Nuclear Engineering and Technology, 2021, 53, 3367-3378.                                | 2.3 | 10        |
| 22 | The TRANSURANUS fuel performance code. , 2021, , 161-205.  |     | 8         |
| 23 | Integrated design of breeding blanket and ancillary systems related to the use of helium or water as a coolant and impact on the overall plant design. Fusion Engineering and Design, 2021, 173, 112933. | 1.9 | 23        |
| 24 | The DEMO Water-Cooled Lead-Lithium Breeding Blanket: Design Status at the End of the Pre-Conceptual Design Phase. Applied Sciences (Switzerland), 2021, 11, 11592.                                       | 2.5 | 54        |
| 25 | Experimental and Numerical Results of LIFUS5/Mod3 Series E Test on In-Box LOCA Transient for WCLL-BB. Energies, 2021, 14, 8527.  | 3.1 | 9         |
| 26 | Design and preliminary analyses of the new Water Cooled Lithium Lead TBM for ITER. Fusion Engineering and Design, 2020, 160, 111921.   | 1.9 | 34        |
| 27 | Parametric study of the influence of double-walled tubes layout on the DEMO WCLL breeding blanket thermal performances. Fusion Engineering and Design, 2020, 161, 111893.                                | 1.9 | 6         |
| 28 | Optimization of the first wall cooling system for the DEMO WCLL blanket. Fusion Engineering and Design, 2020, 161, 111903.   | 1.9 | 8         |
| 29 | Fusion technologies development at ENEA Brasimone Research Centre: Status and perspectives. Fusion Engineering and Design, 2020, 160, 112008.  | 1.9 | 9         |
| 30 | Experimental Characterization of Leak Detection Systems in HLM Pool Using LIFUS5/Mod3 Facility. Nuclear Technology, 2020, 206, 1409-1420.  | 1.2 | 6         |
| 31 | Influence of PbLi hydraulic path and integration layout on MHD pressure losses. Fusion Engineering and Design, 2020, 155, 111517.  | 1.9 | 24        |
| 32 | Test Series D experimental results for SIMMER code validation of WCLL BB in-box LOCA in LIFUS5/Mod3 facility. Fusion Engineering and Design, 2020, 156, 111582.  | 1.9 | 18        |
| 33 | On the impact of the heat transfer modelling approach on the prediction of EU-DEMO WCLL breeding blanket thermal performances. Fusion Engineering and Design, 2020, 161, 112051.                         | 1.9 | 3         |
| 34 | Hybrid 1D + 2D Modelling for the Assessment of the Heat Transfer in the EU DEMO Water-Cooled Lithium-Lead Manifolds. Energies, 2020, 13, 3525.   | 3.1 | 3         |
| 35 | Modelling and assessment of thermal conductivity and melting behaviour of MOX fuel for fast reactor applications. Journal of Nuclear Materials, 2020, 541, 152410.                                       | 2.7 | 16        |
| 36 | Nuclear analysis of the Water cooled lithium lead DEMO reactor. Fusion Engineering and Design, 2020, 160, 111833.  | 1.9 | 17        |

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|----|---|-----|-----------|
| 37 | Electromagnetic coupling phenomena in co-axial rectangular channels. Fusion Engineering and Design, 2020, 160, 111854.  | 1.9 | 10        |
| 38 | Preliminary design of the top cap of DEMO Water-Cooled Lithium Lead breeding blanket segments. Fusion Engineering and Design, 2020, 161, 111884.  | 1.9 | 5         |
| 39 | Analysis of Test D1.1 of the LIFUS5/Mod3 facility for In-box LOCA in WCLL-BB. Fusion Engineering and Design, 2020, 160, 111832.   | 1.9 | 10        |
| 40 | Assessment of DEMO WCLL breeding blanket primary heat transfer system isolation valve absorbed doses due to activated water. Fusion Engineering and Design, 2020, 160, 111999.                        | 1.9 | 8         |
| 41 | Thermal-hydraulic analysis of the DEMO WCLL elementary cell: BZ tubes layout optimization. Fusion Engineering and Design, 2020, 160, 111956.  | 1.9 | 13        |
| 42 | Thermal-hydraulic modeling and analysis of the Water Cooling System for the ITER Test Blanket Module. Fusion Engineering and Design, 2020, 158, 111709.   | 1.9 | 10        |
| 43 | Progress of the conceptual design of the European DEMO breeding blanket, tritium extraction and coolant purification systems. Fusion Engineering and Design, 2020, 157, 111640.                       | 1.9 | 46        |
| 44 | Investigation of the DEMO WCLL Breeding Blanket Cooling Water Activation. Fusion Engineering and Design, 2020, 157, 111697.   | 1.9 | 13        |
| 45 | MHD forced convection flow in dielectric and electro-conductive rectangular annuli. Fusion Engineering and Design, 2020, 159, 111773.   | 1.9 | 13        |
| 46 | Investigation of heat transfer in a steam generator bayonet tube for the development of PbLi technology for EU DEMO fusion reactor. Fusion Engineering and Design, 2020, 159, 111772.                 | 1.9 | 12        |
| 47 | MHD pressure drop estimate for the WCLL in-magnet PbLi loop. Fusion Engineering and Design, 2020, 160, 111830.  | 1.9 | 14        |
| 48 | Preliminary Thermo-Mechanical Design of the Once Through Steam Generator and Molten Salt Intermediate Heat Exchanger for EU DEMO. IEEE Transactions on Plasma Science, 2020, 48, 1726-1732.           | 1.3 | 1         |
| 49 | RELAP5/SIMMER-III code coupling development for PbLi-water interaction. Fusion Engineering and Design, 2020, 153, 111504.   | 1.9 | 16        |
| 50 | RELAP5-3D Three-Dimensional Analysis Based on PHÂNIX Dissymmetric Transient Test. Journal of Nuclear Engineering and Radiation Science, 2020, 6, .  | 0.4 | 0         |
| 51 | Systems engineering activities supporting the heating & current drive and fuelling lines systems integration in the European DEMO breeding blanket. Fusion Engineering and Design, 2019, 147, 111265. | 1.9 | 11        |
| 52 | Parametric study of the influence of First Wall cooling water on the Water Cooled Lithium Lead Breeding Blanket nuclear response. Fusion Engineering and Design, 2019, 146, 2070-2073.                | 1.9 | 5         |
| 53 | Thermal-hydraulic and thermo-mechanical simulations of Water-Heavy Liquid Metal interactions towards the DEMO WCLL breeding blanket design. Fusion Engineering and Design, 2019, 146, 2712-2716.      | 1.9 | 9         |
| 54 | Experimental analysis of stationary and transient scenarios of alfred steam generator bayonet tube in circe-hero facility. Nuclear Engineering and Design, 2019, 352, 110169.                         | 1.7 | 22        |

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| 55 | DEMO WCLL breeding zone cooling system design: Analysis and discussion. Fusion Engineering and Design, 2019, 146, 2632-2638.   | 1.9 | 12        |
| 56 | Post-test simulation of a PLOFA transient test in the CIRCE-HERO facility. Nuclear Engineering and Design, 2019, 355, 110321.  | 1.7 | 16        |
| 57 | Steam Generator mock-up preliminary design suitable for Pb-Li technology demonstration and code assessment. Fusion Engineering and Design, 2019, 146, 1126-1130.   | 1.9 | 6         |
| 58 | Parametric thermal analysis for the optimization of Double Walled Tubes layout in the Water Cooled Lithium Lead inboard blanket of DEMO fusion reactor. Journal of Physics: Conference Series, 2019, 1224, 012031. | 0.4 | 4         |
| 59 | System thermal-hydraulic modelling of the phœnix dissymmetric test benchmark. Nuclear Engineering and Design, 2019, 353, 110272.   | 1.7 | 6         |
| 60 | Experimental activities for in-box LOCA of WCLL BB in LIFUS5/Mod3 facility. Fusion Engineering and Design, 2019, 146, 914-919.   | 1.9 | 26        |
| 61 | Validation of SIMMER-III code for in-box LOCA of WCLL BB: Pre-test numerical analysis of Test D1.1 in LIFUS5/Mod3 facility. Fusion Engineering and Design, 2019, 146, 978-982.                                     | 1.9 | 13        |
| 62 | Investigation on cooling performance of WCLL breeding blanket first wall for EU DEMO. Fusion Engineering and Design, 2019, 146, 2748-2756.   | 1.9 | 8         |
| 63 | Development of a SIMMERRELAP5 coupling tool. Fusion Engineering and Design, 2019, 146, 1993-1997.  | 1.9 | 13        |
| 64 | Status of Pb-16Li technologies for European DEMO fusion reactor. Fusion Engineering and Design, 2019, 146, 2676-2681.  | 1.9 | 21        |
| 65 | Recent progress in developing a feasible and integrated conceptual design of the WCLL BB in EUROfusion project. Fusion Engineering and Design, 2019, 146, 1805-1809.   | 1.9 | 126       |
| 66 | Updated design of water-cooled breeder blanket for CFETR. Fusion Engineering and Design, 2019, 146, 1716-1720.   | 1.9 | 33        |
| 67 | MHD mixed convection flow in the WCLL: Heat transfer analysis and cooling system optimization. Fusion Engineering and Design, 2019, 146, 809-813.  | 1.9 | 30        |
| 68 | On the effect of stiffening plates configuration on the DEMO Water Cooled Lithium Lead Breeding Blanket module thermo-mechanical behaviour. Fusion Engineering and Design, 2019, 146, 2247-2250.                   | 1.9 | 18        |
| 69 | On the effects of the Double-Walled Tubes lay-out on the DEMO WCLL breeding blanket module thermal behavior. Fusion Engineering and Design, 2019, 146, 883-886.  | 1.9 | 3         |
| 70 | Alternative design of DEMO Water Cooled Lithium Lead internal structure. Fusion Engineering and Design, 2019, 146, 1056-1059.  | 1.9 | 7         |
| 71 | Thermal-hydraulic modeling and analyses of the water-cooled EU DEMO using RELAP5 system code. Fusion Engineering and Design, 2019, 146, 1121-1125.   | 1.9 | 26        |
| 72 | System thermal hydraulics for liquid metals. , 2019, , 157-184.  |     | 2         |

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| 73 | Recent Progress in the WCLL Breeding Blanket Design for the DEMO Fusion Reactor. IEEE Transactions on Plasma Science, 2018, 46, 1446-1457.  | 1.3 | 49        |
| 74 | GEN-IV LFR development: Status & perspectives. Progress in Nuclear Energy, 2018, 105, 318-331.  | 2.9 | 91        |
| 75 | Thermo-hydraulic analysis of EU DEMO WCLL breeding blanket. Fusion Engineering and Design, 2018, 130, 48-55.  | 1.9 | 16        |
| 76 | Advancements in DEMO WCLL breeding blanket design and integration. International Journal of Energy Research, 2018, 42, 27-52.   | 4.5 | 77        |
| 77 | Thermal Hydraulic Analysis of the CIRCE-HERO Pool-Type Facility. , 2018, , .  |     | 4         |
| 78 | Parametric thermal-hydraulic analysis of the EU DEMO Water-Cooled Lithium-Lead First Wall using the GETHEM code. Fusion Engineering and Design, 2018, 137, 257-267.                               | 1.9 | 4         |
| 79 | Numerical assessment of the thermomechanical behaviour of the DEMO Water-Cooled Lithium Lead inboard blanket equatorial module. Fusion Engineering and Design, 2018, 136, 1178-1185.              | 1.9 | 8         |
| 80 | Neutronic analyses in support of the WCLL DEMO design development. Fusion Engineering and Design, 2018, 136, 1260-1264.   | 1.9 | 30        |
| 81 | Analysis of EM loads on DEMO WCLL breeding blanket during VDE-up. Fusion Engineering and Design, 2018, 136, 1523-1528.  | 1.9 | 16        |
| 82 | Multi-Module vs. Single-Module concept: Comparison of thermomechanical performances for the DEMO Water-Cooled Lithium Lead breeding blanket. Fusion Engineering and Design, 2018, 136, 1472-1478. | 1.9 | 21        |
| 83 | Pre-test analysis of accidental transients for ALFRED SGBT mock-up characterization. Nuclear Engineering and Design, 2018, 333, 181-195.  | 1.7 | 13        |
| 84 | Status of EU DEMO heat transport and power conversion systems. Fusion Engineering and Design, 2018, 136, 1557-1566.   | 1.9 | 50        |
| 85 | Study of EU DEMO WCLL breeding blanket and primary heat transfer system integration. Fusion Engineering and Design, 2018, 136, 828-833.   | 1.9 | 44        |
| 86 | Thermo-structural design of the European DEMO water-cooled blanket with a multiscale-multiphysics framework. Fusion Engineering and Design, 2018, 135, 31-41.                                     | 1.9 | 10        |
| 87 | Progress in EU Breeding Blanket design and integration. Fusion Engineering and Design, 2018, 136, 782-792.  | 1.9 | 50        |
| 88 | PhÃ©nix Transient Analysis for the Assessment of RELAP5-3D Based on Dissymmetric Test Benchmark. , 2018, , .  |     | 2         |
| 89 | Structural analysis of the back supporting structure of the DEMO WCLL outboard blanket. Fusion Engineering and Design, 2017, 124, 944-947.  | 1.9 | 17        |
| 90 | Dynamic thermal-hydraulic modelling of the EU DEMO WCLL breeding blanket cooling loops. Fusion Engineering and Design, 2017, 124, 887-891.  | 1.9 | 13        |

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| 91  | Rationale and method for design of DEMO WCLL breeding blanket poloidal segmentation. Fusion Engineering and Design, 2017, 124, 664-668.                       | 1.9 | 10        |
| 92  | WCLL breeding blanket design and integration for DEMO 2015: status and perspectives. Fusion Engineering and Design, 2017, 124, 682-686.                       | 1.9 | 91        |
| 93  | Preliminary System Modeling for the EUROfusion Water Cooled Lithium Lead Blanket. Fusion Science and Technology, 2017, 71, 444-449.                           | 1.1 | 15        |
| 94  | Experimental Investigation in LIFUS5/Mod2 Facility of Spiral-Tube Steam Generator Rupture Scenarios for ELFR. , 2017, , .                                     |     | 3         |
| 95  | HERO Test Section for Experimental Investigation of Steam Generator Bayonet Tube of ALFRED. , 2017, , .   |     | 3         |
| 96  | DEMO port plug design and integration studies. Nuclear Fusion, 2017, 57, 116028.  | 3.5 | 6         |
| 97  | Post-test analyses of LIFUS5 Test#3 experiment. Fusion Engineering and Design, 2017, 124, 856-860.  | 1.9 | 14        |
| 98  | Progress in EU-DEMO in-vessel components integration. Fusion Engineering and Design, 2017, 124, 562-566.  | 1.9 | 20        |
| 99  | CFD analysis of WCLL BB PbLi manifold. Fusion Engineering and Design, 2017, 124, 1015-1018.   | 1.9 | 3         |
| 100 | On the thermo-mechanical behaviour of DEMO water-cooled lithium lead equatorial outboard blanket module. Fusion Engineering and Design, 2017, 124, 725-729.   | 1.9 | 24        |
| 101 | CFD simulation of the magnetohydrodynamic flow inside the WCLL breeding blanket module. Fusion Engineering and Design, 2017, 124, 705-709.                    | 1.9 | 20        |
| 102 | Pre-test analysis of protected loss of primary pump transients in CIRCE-HERO facility. Journal of Physics: Conference Series, 2017, 923, 012005.              | 0.4 | 1         |
| 103 | Validation of a Three-Dimensional Model of EBR-II and Assessment of RELAP5-3D Based on SHRT-17 Test. Nuclear Technology, 2016, 193, 1-14.                     | 1.2 | 8         |
| 104 | Assessment of SIMMER-III Code Based on Steam Generator Tube Rupture Experiments in LIFUS5/Mod2 Facility. , 2016, , .  |     | 4         |
| 105 | Experimental Investigation of Spiral Tubes Steam Generator Rupture Scenarios in LIFUS5/Mod2 Facility for ELFR. , 2016, , .                                    |     | 3         |
| 106 | Addressing the heavy liquid metal “ Water interaction issue in LBE system. Progress in Nuclear Energy, 2016, 89, 204-212.                                     | 2.9 | 15        |
| 107 | Optimization of the breeder zone cooling tubes of the DEMO Water-Cooled Lithium Lead breeding blanket. Fusion Engineering and Design, 2016, 109-111, 227-231. | 1.9 | 16        |
| 108 | Implementation of the chemical PbLi/water reaction in the SIMMER code. Fusion Engineering and Design, 2016, 109-111, 468-473.                                 | 1.9 | 24        |

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| 109 | Objectives and status of EUROfusion DEMO blanket studies. Fusion Engineering and Design, 2016, 109-111, 1199-1206.  | 1.9 | 168       |
| 110 | Simulation study of pressure trends in the case of loss of coolant accident in Water Cooled Lithium Lead blanket module. Fusion Engineering and Design, 2015, 98-99, 1763-1766.           | 1.9 | 17        |
| 111 | Experimental investigation and SIMMER-III code modelling of LBE-water interaction in LIFUS5/Mod2 facility. Nuclear Engineering and Design, 2015, 290, 119-126.                            | 1.7 | 36        |
| 112 | Development of a model for the thermal-hydraulic characterization of the He-FUS3 loop. Fusion Engineering and Design, 2015, 96-97, 212-216.   | 1.9 | 8         |
| 113 | MHD issues related to the use of Lithium Lead eutectic as breeder material for blankets of fusion power plants. Magnetohydrodynamics, 2015, 51, 185-194.                                  | 0.3 | 0         |
| 114 | Coupled Simulations of Natural and Forced Circulation Tests in NACIE Facility Using RELAP5 and ANSYS Fluent Codes. , 2014, , .  |     | 6         |
| 115 | Water/Pb-Bi Interaction Experiments in LIFUS5/Mod2 Facility Modelled by SIMMER Code. , 2014, , .  |     | 7         |
| 116 | Qualification of TRACE V5.0 Code against Fast Cooldown Transient in the PKL-III Integral Test Facility. Science and Technology of Nuclear Installations, 2013, 2013, 1-11.                | 0.8 | 4         |
| 117 | Validation of Advanced Computer Codes for VVER Technology: LB-LOCA Transient in PSB-VVER Facility. Science and Technology of Nuclear Installations, 2012, 2012, 1-15.                     | 0.8 | 10        |
| 118 | Consistent Posttest Calculations for LOCA Scenarios in LOBI Integral Facility. Science and Technology of Nuclear Installations, 2012, 2012, 1-16.   | 0.8 | 4         |
| 119 | OECD/NRC PSBT Benchmark: Investigating the CATHARE2 Capability to Predict Void Fraction in PWR Fuel Bundle. Science and Technology of Nuclear Installations, 2012, 2012, 1-10.            | 0.8 | 3         |
| 120 | Integral Test Facilities and Thermal-Hydraulic System Codes in Nuclear Safety Analysis. Science and Technology of Nuclear Installations, 2012, 2012, 1-3.                                 | 0.8 | 6         |
| 121 | Methodology for the Analysis of Fuel Behavior During LOCA and RIA for Licensing Purposes. , 2012, , .   |     | 1         |
| 122 | Assessing Sensitivity of Observations in Source Term Estimation for Nuclear Accidents. , 2012, , .  |     | 1         |
| 123 | Modeling of BWR Inter-Ramp Project experiments by means of TRANSURANUS code. Annals of Nuclear Energy, 2012, 50, 238-250.   | 1.8 | 5         |
| 124 | Investigation of accident management procedures related to loss of feedwater and station blackout in PSB-VVER integral test facility. Nuclear Engineering and Design, 2012, 250, 633-645. | 1.7 | 11        |
| 125 | Preliminary Discussion on LFR Fuel Pin Design: Current Status, Fuel Modeling and Open Issues. , 2012, , .   |     | 2         |
| 126 | Modeling Large Break-LOCA: In Reactor Fuel Bundle Materials Test MT-4 and MT-6A. , 2012, , .  |     | 0         |



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| 127 | Characteristics Analysis of SCWR During Partial Loss of Reactor Coolant Flow Transient. , 2012, , .   |     | 0         |
| 128 | LIFUS5/Mod2: The Experimental Facility for HLM/Water Interaction Investigation. , 2012, , .   |     | 0         |
| 129 | Post Test Analysis of ICE Tests. , 2012, , .  |     | 2         |
| 130 | Experimental investigation of in-vessel mixing phenomena in a VVER-1000 scaled test facility during unsteady asymmetric transients. Nuclear Engineering and Design, 2011, 241, 3068-3075. | 1.7 | 12        |
| 131 | Uncertainty and sensitivity analysis of a LBLOCA in a PWR Nuclear Power Plant: Results of the Phase V of the BEMUSE programme. Nuclear Engineering and Design, 2011, 241, 4206-4222.      | 1.7 | 60        |
| 132 | Capabilities of TRANSURANUS code in simulating power ramp tests from the IFPE database. Nuclear Engineering and Design, 2011, 241, 1078-1086.   | 1.7 | 7         |
| 133 | A Procedure to Address the Fuel Rod Failures during LB-LOCA Transient in Atucha-2 NPP. Science and Technology of Nuclear Installations, 2011, 2011, 1-11.                                 | 0.8 | 8         |
| 134 | Main Results of Phase IV BEMUSE Project: Simulation of LBLOCA in an NPP. Science and Technology of Nuclear Installations, 2010, 2010, 1-9.  | 0.8 | 3         |
| 135 | Post Test Analysis of LB LOCA Transient in PSB-VVER by CATHARE2 Code. , 2010, , .   |     | 0         |
| 136 | A Procedure to Address the Fuel Rod Failures During LB-LOCA Transient in Atucha-2 NPP. , 2010, , .  |     | 0         |
| 137 | Optimizing the Initial Pressure of Accumulators for the Atucha2 NPP Using an Optimization Method. , 2009, , .   |     | 0         |
| 138 | CFD Analysis of a Slug Mixing Experiment Conducted on a VVER-1000 Model. Science and Technology of Nuclear Installations, 2009, 2009, 1-12.   | 0.8 | 8         |
| 139 | Assessment of TRANSURANUS Fuel Performance Code Against Studsvik Inter-Ramp BWR Database. , 2009, , .   |     | 0         |
| 140 | Addressing Boron Dilution Scenario Through RELAP5/3.3 Analysis of PWR SB LOCA. , 2009, , .  |     | 0         |
| 141 | Assessment of 12 CHF prediction methods, for an axially non-uniform heat flux distribution, with the RELAP5 computer code. Nuclear Engineering and Design, 2008, 238, 2718-2725.          | 1.7 | 6         |
| 142 | The Design of PSB-VVER Experiments Relevant to Accident Management. Journal of Power and Energy Systems, 2008, 2, 371-385.  | 0.5 | 4         |