

Peng Hong Liem

List of Publications by Year in descending order

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all docs

38
docs citations

38
times ranked

113
citing authors

#	ARTICLE	IF	CITATIONS
1	Performance indices optimization of live fission products transmutation in fast reactors. International Journal of Energy Research, 2022, 46, 1327-1338.	4.5	3
2	Analysis of the excess reactivity and control rod worth of RSG-GAS equilibrium silicide core using Continuous-Energy Monte Carlo Serpent2 code. Annals of Nuclear Energy, 2021, 154, 108107.	1.8	7
3	Neutron importance based lattice-to-core projection technique and its application for HTGR design using Monte Carlo method. Nuclear Engineering and Design, 2021, 381, 111338.	1.7	2
4	Preliminary investigation on the sodium fast reactor concave cores with near zero or negative sodium void reactivity. International Journal of Energy Research, 2020, 44, 8025-8036.	4.5	0
5	Analysis of OECD/NEA medium 1000 MWth sodium-cooled fast reactor using the Monte Carlo serpent code and ENDF/B-VIII.0 nuclear data library. Nuclear Science and Techniques/Hewuli, 2020, 31, 1.	3.4	0
6	Physics study of block/prismatic-type HTGR design option for the Indonesian Experimental Power Reactor (RDE). Nuclear Engineering and Design, 2020, 368, 110821.	1.7	2
7	Analysis of the first core of the Indonesian multipurpose research reactor RSG-GAS using the Serpent Monte Carlo code and the ENDF/B-VIII.0 nuclear data library. Nuclear Engineering and Technology, 2020, 52, 2725-2732.	2.3	8
8	Benchmarking the new ENDF/B-VIII.0 nuclear data library for the first core of Indonesian multipurpose research reactor (RSG GAS). EPJ Web of Conferences, 2020, 239, 22007.	0.3	2
9	Nuclear data sensitivity and uncertainty analyses on the first core criticality of the RSG GAS multipurpose research reactor. EPJ Web of Conferences, 2020, 239, 22011.	0.3	1
10	Neutronics design of VVER-1000 fuel assembly with burnable poison particles. Nuclear Engineering and Technology, 2019, 51, 1729-1737.	2.3	28
11	Minor Actinide Transmutation in Supercritical-CO ₂ -Cooled and Sodium-Cooled Fast Reactors with Low Burnup Reactivity Swings. Nuclear Technology, 2019, 205, 1460-1473.	1.2	4
12	Sensitivity and uncertainty analysis on the first core criticality of the RSG GAS multipurpose research reactor. Progress in Nuclear Energy, 2019, 114, 46-60.	2.9	11
13	Evaluation on fuel cycle and loading scheme of the Indonesian experimental power reactor (RDE) design. Nuclear Engineering and Design, 2018, 340, 245-259.	1.7	6
14	Kinetics parameters evaluation on the first core of the RSG GAS (MPR-30) using continuous energy Monte Carlo method. Progress in Nuclear Energy, 2018, 109, 196-203.	2.9	15
15	Feasibility of using Gd ₂ O ₃ particles in VVER-1000 fuel assembly for controlling excess reactivity. Energy Procedia, 2017, 131, 29-36.	1.8	8
16	Alternative Fueling Scheme for the Indonesian Experimental Power Reactor (10 MWth Pebble-Bed) Tj ETQq0 0 0 rgBT, /Overlock 10 Tf 50	1.8	9
17	Analysis of NEA-NSC PWR Uncontrolled Control Rod Withdrawal at Zero Power Benchmark Cases with NODAL3 Code. Science and Technology of Nuclear Installations, 2017, 2017, 1-8.	0.8	6
18	NODAL3 Sensitivity Analysis for NEACRP 3D LWR Core Transient Benchmark (PWR). Science and Technology of Nuclear Installations, 2016, 2016, 1-11.	0.8	6

#	ARTICLE	IF	CITATIONS
19	Fuel element burnup measurements for the equilibrium LEU silicide RSG GAS (MPR-30) core under a new fuel management strategy. <i>Annals of Nuclear Energy</i> , 2016, 98, 211-217.	1.8	17
20	Burnup performance of small-sized long-life CANDLE high temperature gas-cooled reactors with U-235 fuel. <i>Annals of Nuclear Energy</i> , 2016, 91, 36-47.	1.8	5
21	Impact of new evaluated nuclear data libraries on core characteristics of innovative reactor designs. <i>Progress in Nuclear Energy</i> , 2016, 87, 74-88.	2.9	2
22	Absolute burnup measurement of LEU silicide fuel plate irradiated in the RSG GAS multipurpose reactor by destructive radiochemical technique. <i>Annals of Nuclear Energy</i> , 2015, 85, 613-620.	1.8	7
23	TRACY and SILENE Benchmark Phase II evaluation by TRACE code. <i>Progress in Nuclear Energy</i> , 2015, 85, 71-82.	2.9	1
24	Neutronic feasibility study of U-235 based high burnup fuel for pebble bed reactors. <i>Progress in Nuclear Energy</i> , 2015, 80, 17-23.	2.9	7
25	Design optimization of a new homogeneous reactor for medical radioisotope Mo-99/Tc-99m production. <i>Progress in Nuclear Energy</i> , 2015, 82, 191-196.	2.9	12
26	ICONE23-1561 USE OF PA-231 FOR AXIAL POWER DISTRIBUTION FLATTENING OF THORIUM FUEL CANDLE HIGH TEMPERATURE GAS-COOLED REACTORS. The Proceedings of the International Conference on Nuclear Engineering (ICONE), 2015, 2015.23, _ICONE23-1-_ICONE23-1.	0.0	1
27	VALIDATION OF FULL CORE GEOMETRY MODEL OF THE NODAL3 CODE IN THE PWR TRANSIENT BENCHMARK PROBLEMS. <i>Tri Dasa Mega</i> , 2015, 17, 141.	0.5	4
28	The Verification of Coupled Neutronics Thermal-Hydraulics Code NODAL3 in the PWR Rod Ejection Benchmark. <i>Science and Technology of Nuclear Installations</i> , 2014, 2014, 1-9.	0.8	6
29	Development of new ORIGEN2 data library sets for research reactors with light water cooled oxide and silicide LEU (20w/o) fuels based on JENDL-3.3 nuclear data. <i>Nuclear Engineering and Design</i> , 2013, 262, 52-62.	1.7	8
30	Nondestructive burnup verification by gamma-ray spectroscopy of LEU silicide fuel plates irradiated in the RSG GAS multipurpose reactor. <i>Annals of Nuclear Energy</i> , 2013, 56, 57-65.	1.8	12
31	Benchmarking the new JENDL-4.0 library on criticality experiments of a research reactor with oxide LEU (20 w/o) fuel, light water moderator and beryllium reflectors. <i>Annals of Nuclear Energy</i> , 2012, 44, 58-64.	1.8	14
32	Design of transition cores of RSG GAS (MPR-30) with higher loading silicide fuel. <i>Nuclear Engineering and Design</i> , 2010, 240, 1433-1442.	1.7	18
33	Small high temperature gas-cooled reactors with innovative nuclear burning. <i>Progress in Nuclear Energy</i> , 2008, 50, 251-256.	2.9	9
34	Development and verification of fuel burn-up calculation model in a reduced reactor geometry. <i>Annals of Nuclear Energy</i> , 2008, 35, 167-174.	1.8	3
35	Long life small CANDLE-HTGRs with thorium. <i>Annals of Nuclear Energy</i> , 2007, 34, 120-129.	1.8	16
36	Study on the control rod interaction effect in RSG gas multipurpose reactor (MPR-30). <i>Annals of Nuclear Energy</i> , 2002, 29, 701-716.	1.8	11

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37	Fuel management strategy for the new equilibrium silicide core design of RSG GAS (MPR-30). Nuclear Engineering and Design, 1998, 180, 207-219.	1.7	25
38	Neutronic and thermal hydraulic design of the graphite moderated helium-cooled high flux reactor. Nuclear Engineering and Design, 1993, 139, 221-233.	1.7	10