Peng Hong Liem

List of Publications by Year
in descending order

38 papers	306 citations	840776 11 h-index	996975 15 g-index
38 all docs	38 docs citations	38 times ranked	113 citing authors

#	Article	IF	Citations
1	Neutronics design of VVER-1000 fuel assembly with burnable poison particles. Nuclear Engineering and Technology, 2019, 51, 1729-1737.	2.3	28
2	Fuel management strategy for the new equilibrium silicide core design of RSG GAS (MPR-30). Nuclear Engineering and Design, 1998, 180, 207-219.	1.7	25
3	Design of transition cores of RSG GAS (MPR-30) with higher loading silicide fuel. Nuclear Engineering and Design, 2010, 240, 1433-1442.	1.7	18
4	Fuel element burnup measurements for the equilibrium LEU silicide RSG GAS (MPR-30) core under a new fuel management strategy. Annals of Nuclear Energy, 2016, 98, 211-217.	1.8	17
5	Long life small CANDLE-HTGRs with thorium. Annals of Nuclear Energy, 2007, 34, 120-129.	1.8	16
6	Kinetics parameters evaluation on the first core of the RSG GAS (MPR-30) using continuous energy Monte Carlo method. Progress in Nuclear Energy, 2018, 109, 196-203.	2.9	15
7	Benchmarking the new JENDL-4.0 library on criticality experiments of a research reactor with oxide LEU (20 w/o) fuel, light water moderator and beryllium reflectors. Annals of Nuclear Energy, 2012, 44, 58-64.	1.8	14
8	Nondestructive burnup verification by gamma-ray spectroscopy of LEU silicide fuel plates irradiated in the RSG GAS multipurpose reactor. Annals of Nuclear Energy, 2013, 56, 57-65.	1.8	12
9	Design optimization of a new homogeneous reactor for medical radioisotope Mo-99/Tc-99m production. Progress in Nuclear Energy, 2015, 82, 191-196.	2.9	12
10	Study on the control rod interaction effect in RSG gas multipurpose reactor (MPR-30). Annals of Nuclear Energy, 2002, 29, 701-716.	1.8	11
11	Sensitivity and uncertainty analysis on the first core criticality of the RSG GAS multipurpose research reactor. Progress in Nuclear Energy, 2019, 114, 46-60.	2.9	11
12	Neutronic and thermal hydraulic design of the graphite moderated helium-cooled high flux reactor. Nuclear Engineering and Design, 1993, 139, 221-233.	1.7	10
13	Small high temperature gas-cooled reactors with innovative nuclear burning. Progress in Nuclear Energy, 2008, 50, 251-256.	2.9	9
14	Alternative Fueling Scheme for the Indonesian Experimental Power Reactor (10 MWth Pebble-Bed) Tj ETQq0 0 0 rgBT /Overlock 10 Tf 50	1.8	9
15	Development of new ORIGEN2 data library sets for research reactors with light water cooled oxide and silicide LEU (20w/o) fuels based on JENDL-3.3 nuclear data. Nuclear Engineering and Design, 2013, 262, 52-62.	1.7	8
16	Feasibility of using Gd 2 O 3 particles in VVER-1000 fuel assembly for controlling excess reactivity. Energy Procedia, 2017, 131, 29-36.	1.8	8
17	Analysis of the first core of the Indonesian multipurpose research reactor RSG-GAS using the Serpent Monte Carlo code and the ENDF/B-VIII.0 nuclear data library. Nuclear Engineering and Technology, 2020, 52, 2725-2732.	2.3	8
18	Absolute burnup measurement of LEU silicide fuel plate irradiated in the RSG GAS multipurpose reactor by destructive radiochemical technique. Annals of Nuclear Energy, 2015, 85, 613-620.	1.8	7

#	Article	IF	Citations
19	Neutronic feasibility study of Uâ€“Thâ€“Pa based high burnup fuel forÂpebble bed reactors. Progress in Nuclear Energy, 2015, 80, 17-23.	2.9	7
20	Analysis of the excess reactivity and control rod worth of RSG-GAS equilibrium silicide core using Continuous-Energy Monte Carlo Serpent2 code. Annals of Nuclear Energy, 2021, 154, 108107.	1.8	7
21	The Verification of Coupled Neutronics Thermal-Hydraulics Code NODAL3 in the PWR Rod Ejection Benchmark. Science and Technology of Nuclear Installations, 2014, 2014, 1-9.	0.8	6
22	NODAL3 Sensitivity Analysis for NEACRP 3D LWR Core Transient Benchmark (PWR). Science and Technology of Nuclear Installations, 2016, 2016, 1-11.	0.8	6
23	Analysis of NEA-NSC PWR Uncontrolled Control Rod Withdrawal at Zero Power Benchmark Cases with NODAL3 Code. Science and Technology of Nuclear Installations, 2017, 2017, 1-8.	0.8	6
24	Evaluation on fuel cycle and loading scheme of the Indonesian experimental power reactor (RDE) design. Nuclear Engineering and Design, 2018, 340, 245-259.	1.7	6
25	Burnup performance of small-sized long-life CANDLE high temperature gas-cooled reactors with Uâ€“Thâ€“Pa fuel. Annals of Nuclear Energy, 2016, 91, 36-47.	1.8	5
26	Minor Actinide Transmutation in Supercritical-CO₂-Cooled and Sodium-Cooled Fast Reactors with Low Burnup Reactivity Swings. Nuclear Technology, 2019, 205, 1460-1473.	1.2	4
27	VALIDATION OF FULL CORE GEOMETRY MODEL OF THE NODAL3 CODE IN THE PWR TRANSIENT BENCHMARK PROBLEMS. Tri Dasa Mega, 2015, 17, 141.	0.5	4
28	Development and verification of fuel burn-up calculation model in a reduced reactor geometry. Annals of Nuclear Energy, 2008, 35, 167-174.	1.8	3
29	Performance indices optimization of l <scp>ongâ€lived</scp> fission products transmutation in fast reactors. International Journal of Energy Research, 2022, 46, 1327-1338.	4.5	3
30	Impact of new evaluated nuclear data libraries on core characteristics of innovative reactor designs. Progress in Nuclear Energy, 2016, 87, 74-88.	2.9	2
31	Physics study of block/prismatic-type HTGR design option for the Indonesian Experimental Power Reactor (RDE). Nuclear Engineering and Design, 2020, 368, 110821.	1.7	2
32	Neutron importance based lattice-to-core projection technique and its application for HTGR design using Monte Carlo method. Nuclear Engineering and Design, 2021, 381, 111338.	1.7	2
33	Benchmarking the new ENDF/B-VIII.0 nuclear data library for the first core of Indonesian multipurpose research reactor (RSG GAS). EPJ Web of Conferences, 2020, 239, 22007.	0.3	2
34	TRACY and SILENE Benchmark Phase II evaluation by TRACE code. Progress in Nuclear Energy, 2015, 85, 71-82.	2.9	1
35	Nuclear data sensitivity and uncertainty analyses on the first core criticality of the RSG GAS multipurpose research reactor. EPJ Web of Conferences, 2020, 239, 22011.	0.3	1
36	ICONE23-1561 USE OF PA-231 FOR AXIAL POWER DISTRIBUTION FLATTENING OF THORIUM FUEL CANDLE HIGH TEMPERATURE GAS-COOLED REACTORS. The Proceedings of the International Conference on Nuclear Engineering (ICONE), 2015, 2015.23, _ICONE23-1-_ICONE23-1.	0.0	1

#	Article	IF	Citations
37	Preliminary investigation on the sodium fast reactor concave cores with nearâ€zero or negative sodium void reactivity. International Journal of Energy Research, 2020, 44, 8025-8036.	4.5	0
38	Analysis of OECD/NEA medium 1000 MWth sodium-cooled fast reactor using the Monte Carlo serpent code and ENDF/B-VIII.0 nuclear data library. Nuclear Science and Techniques/Hewuli, 2020, 31, 1.	3.4	0